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Assessment of Radiation Exposure of Astronauts in Space

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105	alpha particles and heavy ions ($2 < Z \leq 28$).	
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PREFACE

Space flight is one of the occupational exposures from natural sources already listed in ICRP Publication 60 (1991), but the number of persons involved in space flights was small and hence no guidelines have been developed by ICRP to date.

The present report is the first publication of the International Commission on Radiological Protection (ICRP) dealing with the topic of radiation exposures of astronauts in space. The following terms of reference were given to the Task Group preparing this report:

- Analyze the radiation fields in space with special attention to the high-LET components and solar particle events,
- Investigate the application of dose quantities used in radiological protection ,
- Describe devices for measurement of particle fluences and doses in space environment,
- Describe procedures of the assessment of doses to astronauts performed by measurements and calculations, and
- Present some data of fluence-to-dose conversion coefficients for heavy ions.

The report was prepared by a Task Group on “Radiation Protection in Space” of ICRP Committee 2. The membership of the Task Group was:

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MAIN POINTS

- The radiation field astronauts are exposed to in space is very complex. It contains electrons, protons, alpha particles and heavier ions up to very high energies from galactic cosmic radiation, protons and electrons emitted from the Sun, protons and electrons trapped in belts by the magnetic field of the Earth, and various secondary radiations created in interactions of the primary radiation with materials of the spacecraft. Depending on the time present in space, mission doses to astronauts may become much higher than 100 mSv. Only external radiation exposure to astronauts needs to be considered.
- Due to the specifics of the radiation field in space not all concepts of quantities defined for radiological protection applications on Earth are appropriate for applications in space missions, especially when risk assessment is an important task. A radiation weighting factor $w_R = 20$ for all types and energies of heavy ions in the definition of equivalent dose is not justified. For heavy ions the ratio of w_R and mean quality factors averaged over the human body is up to a factor of 10 depending on the type and energy of the ion.
- No specific operational dose quantity is recommended for area monitoring in space. Quantities to be measured are radiation fluence rates and energy distributions of different types of particles, and LET-distributions. For individual monitoring, measurement of absorbed dose at the surface of the body in combination with LET-distributions may be appropriate for an assessment of effective dose equivalent or organ dose equivalent.
- Radiation transport calculations are important tools for information about radiation exposure of astronauts. Based on data about the primary radiation fields transport calculations are able to calculate radiation fields inside spacecrafts, fields on the body of astronauts and in organs or tissues of anthropomorphic phantoms.
- Generally, two different procedures may be applied for the assessment of doses in the astronaut's body by calculations. One may either assess the radiation field parameters near to an astronaut and then apply fluence-to-dose conversion coefficients for all types of particles involved for the assessment of organ doses or one may calculate organ doses in a body using the radiation field data outside of the spacecraft and a code which combines radiation transport in the spacecraft and in the human body. Measurements with individual dosimeters near to the body of an astronaut in combination with calculations may be directly correlated to doses in the human body.
- The report presents conversion coefficients for mean absorbed doses in organs and tissues of the body for heavy ions up to $Z = 28$. The calculations are based on the reference voxel phantoms following the 2007 Recommendations of the Commission. Data are given for isotropic (ISO) exposure of the body. Also mean quality factors, Q_T , for the specified organs and tissues of the body are presented. Q_T -values vary between about 2 and 25.
- Information and data are also presented for the quality factor concept derived from the track structure of charged particles in tissue as proposed by NASA... This results in a different distribution of Q in terms of LET with a stronger decrease of Q with increasing LET.

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EXECUTIVE SUMMARY

190 a) Astronauts are living and working in low Earth orbits for extended periods of time,
191 especially during the operation and maintenance of the International Space Station (ISS) and
192 scientific investigations. Furthermore, plans are already discussed for outer space missions of
193 astronauts.

194 b) . In ICRP 103 it is stated that “in exceptional cases of cosmic radiation exposures, such
195 as exposure in space travel, where doses may be significant and some type of control
196 warranted, should be dealt with separately from the conventional approach of occupational
197 exposure”. Therefore, although astronauts are exposed to ionizing radiation during their
198 occupational activities they are usually not classified as being occupationally exposed in the
199 sense of the ICRP system for radiation protection.

200 c) The report contains 7 Chapters. The first one is an introduction describing the specific
201 situation of astronauts in space and the differences of the radiation field in space compared to
202 fields on Earth, thereby showing areas where approaches applied in radiological protection
203 measures on Earth need to be modified.

204 d) In Chapter 2 the radiation fields in space are described in detail. The solar system with
205 the Sun at its centre is embedded in a complex mixture of galactic cosmic radiation (GCR) -
206 protons, α -particles and heavy ions - which continuously enters the heliosphere from all
207 directions. Inside the heliosphere, the GCR fluence rate and particle energy distributions are
208 modulated by the interplanetary magnetic field produced by the charged particles
209 continuously emitted by the Sun, the so-called solar wind. In addition to the solar wind, the
210 Sun occasionally emits unusually large pulses of energetic particles – mostly protons and
211 electrons – called solar-particle events (SPEs). Celestial bodies equipped with a magnetic
212 moment like the Earth are surrounded by toroidal belts of particulate radiation. Such radiation
213 belts constitute an important third primary exposure source. Fluence rates of cosmic radiation
214 are not constant; they vary between two extremes which correspond in time with the
215 maximum and minimum solar activity. Solar activity and cosmic radiation fluence rates are
216 inversely correlated. In Sections 2.2 to 2.5 the various components of the radiation field in
217 space are presented and the influence of the Earth’s magnetic field is described.

218 e) Chapter 3 is dealing with the quantities used in radiological protection. In the first part
219 the system of dose quantities as given in Publication 103 (2007) is described and secondly the
220 relative biological effectiveness (RBE) is discussed especially with respect to the large
221 contribution of heavy ions and the very high energies. A single w_R -value of 20 for all heavy
222 ions of all energies is not appropriate for space radiation fields. Hence, for space applications
223 the concept of a quality factor, $Q(L)$, is applied also to the protection quantities. In Section
224 3.3 the approach for applications in space is described in detail.

225 f) In Chapter 4 the methods of fluence and dose measurements in space are described.
226 This includes instrumentation for fluence measurements, radiation spectrometry, area
227 dosimetry, and individual monitoring. Passive and active devices are mentioned and also the
228 use of biomarkers for the assessment of mission doses is described. Some advice for quality
229 control and the assessment of uncertainties is also given in this Chapter.

230 g) In Chapter 5 the methods of determining quantities describing the radiation fields
231 within a spacecraft are given. Radiation transport calculations are the most important tool for
232 an assessment of radiation fields inside a spacecraft starting from the radiation field in free
233 space and considering the walls and further equipment of the spacecraft. In this chapter some

234 physical data used in radiation transport codes are presented and the various codes used for
235 calculations in high-energy radiation fields as in space described. Results of calculations of
236 radiation fields in spacecrafts are given. A discussion of shielding possibilities is included in
237 this Chapter, too.

238 h) Chapter 6 is dealing with methods of determining mean absorbed doses and dose
239 equivalent s in organs and tissues of the human body. Calculated conversion coefficients of
240 fluence to mean absorbed dose in an organ or tissue are given for heavy ions up to $Z=58$ for
241 energies from 10 MeV/u to 100 GeV/u. For the same set of ions and ion energies mean
242 quality factors in organs and tissues are presented using on the one hand the $Q(L)$ function
243 defined in Publication 60 of the Commission and on the other hand a $Q(Z,E)$ function
244 proposed by NASA. In Sections 6.4 assessment of doses in the body by measurements are
245 described and results are compared with calculations. In Section 6.5 biodosimetric
246 measurements for the assessment of mission doses are presented.

247 i) In Chapter 7 operational measures with regard to the assessment of the exposure of
248 astronauts during space missions. This includes pre-flight mission design, area and individual
249 monitoring during flights in space and dose recording. The importance of considering
250 uncertainties in dose assessment is also mentioned.

251 j) In an Annex numerical data of conversion coefficients and mean quality factors for
252 protons, neutrons, alpha particles and heavy ions ($2 < Z \leq 28$) are given.

253
254

255 GLOSSARY

256 Absorbed dose, D 257 The absorbed dose is the quotient of $d\bar{\varepsilon}$ by dm :

258
$$D = \frac{d\bar{\varepsilon}}{dm}$$

259 where $d\bar{\varepsilon}$ is the mean energy imparted by ionising radiation to matter of mass dm . The unit
260 of absorbed dose is joule per kilogram (J/kg), and its special name is gray (Gy).

261

262 Ambient dose equivalent, $H^*(10)$ 263 The dose equivalent at a point in a radiation field that would be produced by the
264 corresponding expanded and aligned field in the ICRU sphere at depth of 10 mm on the
265 radius vector opposing the direction of the aligned field. The unit of ambient dose equivalent
266 is joule per kilogram (J/kg), and its special name is sievert (Sv).

267

268 Biomarker

269 A traceable substance indicating changes in a cell or an organ caused by environmental
270 actions, e.g. by ionising radiation.

271

272 Charged Particle Equilibrium

273 Charged particles equilibrium in a volume of interest means that the energies, numbers and
274 directions of the charged particles are constant throughout this volume. This is equivalent to
275 saying that the distribution of charged particle energy radiance does not vary within the
276 volume. In particular, it follows that the sums of the energies (excluding rest energies) of the
277 charged particle entering and leaving the volume are equal

278

279 Cross section, σ 280 The cross section of a target entity for a particular interaction produced by incident charged
281 or uncharged particles of a given type and energy is given by:

282
$$\sigma = \frac{N}{\Phi}$$

283 where N is the mean number of such interactions per target entity subjected to the particle
284 fluence, Φ . The unit of cross section is m^2 . A special unit often used for the cross section is
285 the barn where 1 barn (b) = 10^{-28}m^2 . A full description of an interaction process requires,
286 *inter alia*, the knowledge of the distributions of cross sections in terms of energy and
287 direction of all emergent particles from the interaction. Such distributions, sometimes called
288 'differential cross sections', are obtained by differentiations of σ with respect to energy and
289 solid angle.

290

291 Deterministic effect

292 See 'Tissue reaction'.

293

294 Detriment

295 The total harm to health experienced by an exposed group and its descendants as a result of
296 the group's exposure to a radiation source. Detriment is a multi-dimensional concept; its

297 principal components are the stochastic quantities probability of attributable fatal cancer,
298 weighted probability of attributable non-fatal cancer, weighted probability of severe heritable
299 effects, and length of life lost if the harm occurs.

300

301 Directional dose equivalent, $H'(d, \Omega, \square)$

302

303 The dose equivalent at a point in a radiation field that would be produced by the
304 corresponding expanded field in the ICRU sphere at a depth, d , on a radius in a specified
305 direction, Ω . The unit of directional dose equivalent is joule per kilogram (J/kg) and its
306 special name is sievert (Sv).

307

308 Dose conversion coefficient

309 A coefficient relating a dose quantity to a physical quantity, both for internal and external
310 radiation exposure. In internal dosimetry this term is also called a 'dose coefficient'.

311

312 Dose equivalent, H

313 The dose equivalent at a point in tissue is given by:

314

$$H = Q D$$

315 where D is the absorbed dose and Q is the quality factor at that point. The unit of dose
316 equivalent is joule per kilogram (J/kg), and its special name is sievert (Sv).

317

318 Dose equivalent in an organ or tissue, $H_{T,Q}$

319 The product of the mean quality factor, Q_T , and the mean absorbed dose, D_T , in an organ or
320 tissue T: $H_{T,Q} = Q_T D_T$.

321 A quantity introduced by ICRP in Publication 26 and replaced by equivalent dose in an organ
322 or tissue in Publication 60.

323

324

325 Dose limit

326 Recommended value of a dose to an individual that shall not be exceeded in planned
327 exposure situations.

328

329 Effective charge number, Z^*

330 The charge number Z of a nuclei reduced by a factor depending on the relative velocity β
331 (velocity of the nuclei relative to the velocity of light) which takes account of the fact that at
332 low velocities the nuclei is not completely ionised. It is

333

$$Z^* = Z (1 - \exp(-125 \beta Z^{2/3}))$$

334

335 Effective dose, E

336 The tissue-weighted sum of equivalent dose in an organ or tissue from all specified organs
337 and tissues of the body, given by the expression:

338

$$E = \sum_T w_T \sum_R w_R D_{T,R} = \sum_T w_T H_T$$

339 where H_T is the equivalent dose in an organ or tissue T, $D_{T,R}$ is the mean absorbed dose in an
340 organ or tissue T from radiation of type R, and w_T is the tissue weighting factor. The sum is
341 performed over organs and tissues considered to be sensitive to the induction of stochastic

342 effects. The unit of effective dose is joule per kilogram (J/kg), and its special name is sievert
343 (Sv).

344

345 Effective dose equivalent, H_E

346 The tissue-weighted sum of dose equivalent in an organ or tissue from all specified organs
347 and tissues of the body, given by the expression:

$$348 \quad H_E = \sum_T w_T H_{T,Q}$$

349 where $H_{T,Q}$ is the dose equivalent in an organ or tissue T. The unit of effective dose
350 equivalent is joule per kilogram (J/kg), and its special name is sievert (Sv).

351 A quantity introduced by ICRP in Publication 26 and replaced by effective dose in
352 Publication 60.

353

354 Equivalent dose in an organ or tissue, H_T

355 The equivalent dose in an organ or tissue is given by:

$$356 \quad H_T = \sum_R w_R D_{T,R}$$

357 where $D_{T,R}$ is the mean absorbed dose from radiation of type R in the specified organ or
358 tissue T, and w_R is the radiation weighting factor. The unit of equivalent dose is joule per
359 kilogram (J/kg) and its special name is sievert (Sv).

360

361 Fluence, Φ

362 The quotient of dN by da , where dN is the number of particles incident on a sphere of cross-
363 sectional area da , thus:

$$364 \quad \Phi = \frac{dN}{da}$$

365 The unit of fluence is m^{-2} .

366

367 Galactic cosmic radiation (GCR)

368 Charged particle radiation which continuously enters the heliosphere from outer space from
369 all directions. The radiation contains electrons (about 2%) and a broad range of charged
370 particles from protons up to high-Z particles with high energies up to several hundreds of
371 GeV and even higher. The fluence rate of the GCR in the heliosphere is inversely correlated
372 with the solar activity. It is caused by the solar magnetic field, which is coupled to the solar
373 wind.

374

375 Geomagnetic cut-off rigidity

376 See "rigidity threshold".

377

378 Heavy ion

379 Ions of elements heavier than He.

380

381 Heliosphere

382 The space formed by the Sun and its planets.

383

384 ICRU 4-element tissue

385 ICRU 4-element tissue has a density of 1 g/cm³, and a mass composition of 76.2 % oxygen,
386 11.1 % carbon, 10.1 % hydrogen, and 2.6 % nitrogen.

387

388 ICRU sphere

389 A hypothetical sphere, 30 cm in diameter, of tissue-equivalent material (ICRU 4-element
390 tissue) with a density of 1 g cm⁻³ and a mass composition of 76.2 % oxygen, 11.1 % carbon,
391 10.1 % hydrogen and 2.6 % nitrogen.

392

393 K-, Kp-index

394 Index quantifying disturbances in the horizontal component of Earth's magnetic field with a
395 number in the range 0-9 with 1 being calm and 5 or more indicating a geomagnetic storm.
396 The K-index is derived from the maximum fluctuations of horizontal components observed
397 on a magnetometer during a three-hour interval. The official planetary Kp-index is derived by
398 calculating a weighted average of K-indices from a network of geomagnetic observatories.

399

400 Kerma, *K*

401 Quantity for ionising uncharged particles, defined by the quotient of dE_{tr} by dm , where dE_{tr}
402 is the mean sum of the initial kinetic energies of all the charged particles liberated in a mass
403 dm of a material by the uncharged particles incident on dm , thus:

$$404 \quad K = \frac{dE_{tr}}{dm}$$

405 The unit of kerma is joule per kilogram (J/kg) and its special name is gray (Gy).

406

407 Kerma approximation

408 Kerma is sometimes used as an approximation to absorbed dose. The numerical value of the
409 kerma approaches that of the absorbed dose to the degree that charged-particle equilibrium
410 exists, that radiative losses are negligible, and that the kinetic energies of the uncharged
411 particles are large compared to the binding energies of the liberated charged particles.

412

413 Lineal energy

414 The quotient of ε_i by \bar{l} , where ε_i is the energy imparted to the matter in a given volume by
415 a single energy-deposition event i and \bar{l} is the mean chord length of that volume, thus

$$416 \quad y = \frac{\varepsilon_i}{\bar{l}}$$

417 The unit of lineal energy is joule per meter (J/m)

418

419 Linear energy transfer/unrestricted linear energy transfer, *L* or LET

420 The quotient of dE by dl , where dE is the mean energy lost by the charged particle due to
421 electronic interactions in traversing a distance dl thus:

$$422 \quad L = \frac{dE}{dl}$$

423 The unit of linear energy transfer is joule per metre (J/m), often given in keV/ μ m.

424

425 Linear-non-threshold (LNT) model

426 A dose-response model which is based on the concept that, in the low dose range, radiation
427 doses greater than zero will increase the risk of excess cancer and/or heritable disease in a
428 simple proportionate manner.

429

430 Low Earth Orbit (LEO)

431 Orbit in space near to the Earth where astronauts are protected against parts of particles of
432 galactic and solar origin by the Earth magnetic field. The shielding depends on the inclination
433 of the spacecraft. On the other hand the Earth magnetic field is responsible for the formation
434 of the trapped radiation belts where mainly protons and electrons from galactic and solar
435 origin are stored.

436

437 Magnetosphere

438 The Earth magnetic dipole field which extends over a distance from Earth up to about
439 75 000 km around the geomagnetic equator. Charged particles in a specific energy range are
440 trapped in the magnetic field. They move in spirals along the geomagnetic field lines, are
441 reflected back between the magnetic poles acting as mirrors and form radiation belts. These
442 belts were discovered by van Allen.

443

444

445 Mean absorbed dose in an organ or tissue, D_T

446 The mean absorbed dose in a specified organ or tissue T, is given by

447
$$D_T = \frac{1}{m_T} \int D dm \text{ ,}$$

448 where m_T is the mass of the organ or tissue T, and D is the absorbed dose in the mass element
449 dm . The unit of mean absorbed dose is joule per kilogram (J/kg), and its special name is gray
450 (Gy).

451

452 Nuclear track detectors (NTD)

453 Detectors where the tracks of passing charged particles can afterwards be made visible by
454 suitable chemical treatment and microscopically viewed. These passive detectors are either
455 plastic nuclear etched track detectors (PNTD) where the tracks are made visible by chemical
456 etching or nuclear emulsions. Etched track detectors are generally insensitive for radiation
457 with an LET in water below about 10 keV μm^{-1} .

458

459 Occupational exposure

460 The radiation exposure of workers incurred as a result of their work. The Commission limits
461 its use of 'occupational exposures' to radiation exposures incurred at work as a result of
462 situations that can reasonably be regarded as being the responsibility of the operating
463 management.

464

465 Operational quantities

466 Quantities used in practical applications for monitoring and investigating situations
467 involving external exposure and intakes of radionuclides. They are defined for measurements
468 and assessment of doses in the body.

469

470 Organ absorbed dose

471 Short phrase for "mean absorbed dose in an organ or tissue".

472

473 Organ dose equivalent

474 Short phrase for “dose equivalent in an organ or tissue”.

475

476 Organ equivalent dose

477 Short phrase for “equivalent dose in an organ or tissue”.

478

479 Optically stimulated luminescence dosimeters (OSLD)

480 Detectors where the energy stored by particles in the detector material (glasses) is release as
481 luminescence light by optical laser stimulation. The amount of emitted light is proportional to
482 the absorbed dose.

483

484 Personal dose equivalent, $H_p(d)$ 485 The dose equivalent in soft tissue at an appropriate depth, d , below a specified point on the
486 human body. The soft tissue is ICRU 4-element tissue. The unit of personal dose equivalent
487 is joule per kilogram (J/kg) and its special name is sievert (Sv). The specified point is usually
488 given by the position where the individual dosimeter is worn. For the assessment of effective
489 dose, a depth of 10 mm is recommended, and for the assessment of equivalent dose to the
490 skin and the lens of the eye, depths of 0.07 mm and 3 mm, respectively, are recommended.

491

492 Protection quantities

493 Dose quantities related to the human body used for setting exposure limits and in the
494 context of optimisation. They have been developed by the Commission for radiological
495 protection to allow quantification of the detriment to people from exposure of the human
496 body to ionising radiation from both whole and partial body external irradiation and from
497 intakes of radionuclides.498 Quality factor, Q 499 A dimensionless factor defined to reflect the relative biological effectiveness of high-LET
500 radiations compared to low-LET radiation at low exposure levels.501 Q is usually given by a function $Q(L)$, where L is the unrestricted linear energy transfer in
502 water.

503 The quality factor at a point in tissue, is given by

504
$$Q = \frac{1}{D} \int_{L=0}^{\infty} Q(L) D_L dL$$

505 where D is the absorbed dose at that point, D_L is the distribution of D in unrestricted linear
506 energy transfer L at the point of interest, and $Q(L)$ is the quality factor as a function of L . The
507 integration is to be performed over D_L , due to all charged particles, excluding their secondary
508 electrons.

509

510 Radiation belt

511 See “Magnetosphere”.

512

513 Radiation weighting factor, w_R 514 A dimensionless factor by which the mean absorbed dose in an organ or tissue, D_T , is
515 multiplied to reflect the relative biological effectiveness of high-LET radiations compared

516 with low-LET radiations. The product of w_R and D_T is the equivalent dose in the organ or
517 tissue T.

518

519 Reference male and reference female (reference individual)

520 An idealised male or female with characteristics defined by the Commission for the purpose
521 of radiological protection, and with the anatomical and physiological characteristics defined
522 in Publication 89 (ICRP, 2002).

523

524 Reference person

525 An idealised person for whom the equivalent doses in organs and tissues are calculated by
526 averaging the corresponding doses of Reference Male and Reference Female. The equivalent
527 doses of the Reference person are used for the calculation of the effective dose.

528

529 Reference phantom

530 The computational phantom of the human body (male or female voxel phantom based on
531 medical imaging data) defined in Publication 110 (ICRP, 2009) with the anatomical and
532 physiological characteristics defined in Publication 89 (ICRP, 2002).

533 Reference value

534 Value of a quantity recommended by the Commission for use in dosimetric applications or
535 biokinetic models. Reference values are fixed and specified with no uncertainty,
536 independently of the fact that the basis of these values may include many uncertainties.

537

538 Relative biological effectiveness (RBE)

539 The ratio of absorbed dose of a low-LET reference radiation to absorbed dose of the
540 radiation considered that gives an identical biological effect. RBE values vary with absorbed
541 dose, absorbed dose rate and the biological endpoint considered. In radiological protection
542 the RBE at low and very low doses (RBE_M) is especially of interest.

543

544 Rigidity, R

545 The magnetic rigidity of an ion given by its momentum (often given in units of GeV/c)
546 divided by its charge. Parameter used for characterizing the movement of a high-energy
547 charged particle in the magnetic field in space.

548

549 Rigidity threshold (geomagnetic cut-off rigidity), R_c

550 For each point inside the Earth's magnetosphere and each direction from that point there
551 exist a rigidity threshold below which the cosmic particles are not able to reach this point.
552 This rigidity is called the geomagnetic cut-off rigidity, R_c .

553

554 Solar cosmic radiation (SCR)

555 Radiation emitted from the Sun. The radiation includes continuously emitted electrons and
556 protons (solar wind) and also high energy particles (mainly electrons and protons) emitted
557 during solar particle events (SPE).

558

559 Solar cycle

560 Variation of the solar activity between two extremes with a cycle time of about 11 years.
561 The solar activity can be described by the number of observed sunspots.

562

563 Solar particle event (SPE)

564 An eruption at the Sun surface that releases a large number of particles (mostly electrons
565 and protons with few helium ions and heavy charged particles) over the course of hours or
566 days.

567

568 Solar wind

569 Mostly low-energy electrons and protons continuously emitted from the Sun into the
570 heliosphere and producing the interplanetary magnetic field. The intensity of the solar wind
571 depends on solar activity and varies with the solar cycle.

572

573 South Atlantic anomaly (SAA)

574 An area where the radiation belt, and hence the trapped protons, comes closer to the Earth
575 surface due to a displacement of the magnetic dipole axes from the Earth's centre.

576

577 Stochastic effect

578 Effects resulting from damage in a single cell, such as cancer and heritable effects. The
579 frequency of the event, but not its severity, increases with an increase in the dose. For
580 protection purposes it is assumed that there is no threshold dose.

581

582 Thermoluminescence detector (TLD)

583 Small sintered chips or pellets of crystals which show luminescence during heating after
584 exposure to ionising radiation. By fast controlled heating of the crystal, the stored energy is
585 released through light emission. The function between the actual temperature and the
586 intensity of the emitted light (glow curve) shows various peaks, the heights of which are
587 proportional to absorbed dose. These detectors are often used for personal dosimetry.

588

589 Tissue reaction

590 Injury in populations of cells, characterized by a threshold dose and an increase in the
591 severity of the reaction as the dose is increased further, also termed 'deterministic effect'. In
592 some cases, these effects are modifiable by post-irradiation procedures including biological
593 response modifiers.

594

595 Tissue weighting factor, w_T

596 The factor by which the equivalent dose in an organ or tissue T is weighted to represent the
597 relative contribution of that organ or tissue to overall radiation detriment from stochastic
598 effects. It is defined such that:

599
$$\sum_T w_T = 1 .$$

600

601 Trapped particles

602 Particles trapped in the Earth's magnetic field as a result of the interaction of galactic
603 cosmic radiation and solar cosmic radiation with the Earth's magnetic field and the
604 atmosphere. Mainly protons and electrons are involved (see also "Magnetosphere").

605

606 Voxel phantom

607 Computational anthropomorphic phantom based on medical tomographic images in which
608 the anatomy is described by small three-dimensional volume elements (voxels). Collections
609 of these voxels are used to specify the organs and tissues of the human body.



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625

1. INTRODUCTION

626 (1) The radiation field in space is very different from environmental radiation fields on
627 Earth both with respect to the various types of radiation involved and to their intensities. The
628 primary radiation field on the Earth surface is composed of low-LET radiations with small
629 high-LET components including neutrons from cosmic radiation and α -particles emitted from
630 terrestrial radionuclides. In contrast, the primary radiation field in space includes electrons,
631 protons, neutrons, α -particles and heavy ions up to very high energies. Additional secondary
632 radiations (e.g. gamma radiation, electrons, muons, neutrons, pions, and collision and
633 projectile fragments) are produced by interactions within the materials of a spacecraft and its
634 equipment and the astronauts themselves.

635 (2) The term "space" generally means the galactic space outside of the aviation altitudes
636 in the lower atmosphere of the Earth. In manned space flight astronauts may experience
637 three different exposure conditions. The first situation is in low Earth orbit (LEO) where they
638 are protected against parts of particles of galactic and solar origin depending on the
639 inclination of the spacecraft by the Earth magnetic field. On the other hand this magnetic
640 field is responsible for the formation of the trapped radiation belts. In addition, through
641 interaction of the solar and galactic particles with the nuclei of the Earth's atmosphere albedo
642 radiation is created. Secondly, leaving the Earth magnetic field to outer space in
643 interplanetary missions, the radiation exposure due only to particles of solar and galactic
644 origin, which are directly impinge on the spacecraft. In planetary missions, about one
645 hemisphere is shielded by the mass of the planet. If there is no atmosphere, the primary
646 radiation interacts with the nuclei of the soil which leads to the production of secondary
647 particles with a high contribution of high-LET components. If a thin atmosphere is present, as
648 in case of Mars, both interactions in the atmosphere and in the soil contribute to secondary
649 radiation.

650 (3) Astronauts are living and working in low Earth orbits for extended periods of time
651 and will be involved in outer space missions. They are living under extraordinarily different
652 environmental conditions than ever encountered on Earth. For missions outside the
653 magnetosphere, ionising radiation is recognized as the key factor through its impact on crew
654 health and performance. Obviously the radiation environment is quite different from that on
655 Earth and human exposure in space is much higher than on Earth and cannot be avoided by
656 radiation shielding. The reasons for that are the extreme high energies of particles in space
657 radiation fields and their high penetration depth in matter combined with the release of
658 secondary radiations (e.g. neutrons and photons) in interactions of the primary radiation with
659 that material. The exposure of astronauts in space is a special case of environmental
660 exposure. In long term missions, the exposure of astronauts will be higher than the annual
661 limits recommended for exposure of workers on Earth. In ICRP 103 it is stated that "in
662 exceptional cases of cosmic radiation exposures, such as exposure in space travel, where
663 doses may be significant and some type of control warranted, should be dealt with
664 separately". Therefore, although astronauts are exposed to ionizing radiation during their
665 occupational activities they are usually not classified as being occupationally exposed in the
666 sense of the ICRP system for radiation protection. Their situation during space missions is
667 similar to special planned exposure situations in emergency exposure situations, in which so-
668 called rescuers may be exposed to radiation levels during their intervention exceeding dose
669 limits for occupationally exposed persons. Thus, for a specific mission planned, reference
670 values for risks or doses may be selected at appropriate levels and no dose limits may be

671 applied for the given mission.

672 (4) During the last two decades there was a marked development in the activities in
673 space including an increase of the number of astronauts participating in space missions.
674 Nevertheless, even today, the number of astronauts is small as compared to the large number
675 of occupationally exposed persons on Earth and in civil aviation. However, considering the
676 extraordinary exposure situation of this group, radiological protection concepts need to be
677 well defined and realistically implemented with respect to the specific situation found in the
678 space environment and during long-term space missions.

679 (5) The basis for any measure in radiological protection should always be the
680 knowledge of the radiation fields involved. Therefore, measurements of the environmental
681 radiation and the assessment of the exposure of astronauts are very important tasks. Since the
682 discovery of cosmic radiation at the start of the last century by V.F. Hess in 1912 (see e. g.
683 Compton, 1936) the study of cosmic radiation and its various components has already been
684 performed for a long time and has become even more important during the last fifty years
685 when activities in space are strongly increasing and frequently include the presence of
686 astronauts. Obviously the basic information regarding cosmic radiations and their various
687 components can only be obtained through measurements and this has been performed for
688 many years. The specific environmental situation in and around a spacecraft can be estimated
689 either by various measurements at different positions in the specific spacecraft or also by
690 radiation transport calculations when the spacecraft design is sufficiently modelled and the
691 specific composition of the external radiation field including its variation in time is well
692 considered within the simulation code applied. Some parts of this report, therefore, deal with
693 the various components of the radiation field in space (Chapter 2) and with the computer
694 codes and calculational approaches used to simulate both radiation shielding possibilities for
695 the various radiation fields incident upon the spacecraft and the resulting tissue absorbed
696 doses imparted to the astronauts within the spacecraft (Chapters 5 and 6).

697 (6) The specific radiation field in space with its important contribution of heavy ions
698 does not allow simple application of the complete system of dosimetric quantities defined for
699 use in radiological protection on Earth. The radiation weighting factor of 20 defined for all
700 heavy ions of all energies is not appropriate, nor is the concept of the operational dose
701 quantities for external exposure situations applicable to the space situation because very high-
702 energy particles are involved. The concept of operational quantities has been introduced by
703 the ICRU and ICRP mainly looking at electron, photon and neutron radiations of energies up
704 to few tens of MeV and has not considered radiation fields in space which include many
705 other particle types with even higher energies. This topic will be discussed in more detail in
706 Chapter 3.

707 (7) On Earth, radiological protection of workers and the primary dose limits defined are
708 aimed at limiting the probability of the occurrence of stochastic effects, e.g. risk of cancer or
709 hereditary effects, to a level acceptable when compared to other health risks during human
710 life, while at the same time avoiding detriments in humans by deterministic effects (tissue
711 reactions). For practical reasons the primary limits are defined in terms of doses (effective
712 dose and equivalent dose to the skin, hands, feet and lens of the eye where specific limits
713 have been defined for avoiding deterministic effects) which can be assessed with sufficient
714 precision for applications in radiological protection, and not in terms of radiation risks the
715 value of which depends on many individual factors (e.g. age, sex, individual genetic
716 properties). Especially at low levels of exposure, the knowledge regarding these risks is very
717 limited and combined with high uncertainties. The value of the quantity effective dose is
718 calculated by averaging organ equivalent doses over both sexes and using mean values of

719 weighting factors obtained from epidemiological data, hence from large groups of exposed
720 and unexposed persons. Effective dose should, therefore, not be used for the assessment of
721 individual risks. In addition to “limitation” of doses and risks the principle of ALARA (As
722 Low As Reasonably Achievable) is generally applied in radiological protection which means
723 that even below exposure limits “optimisation” of radiation protection always needs to be
724 considered and may require further measures.

725 (8) The situation in space is quite different. An exposure of astronauts by environmental
726 radiation cannot be avoided in space. Prevention by shielding cannot be completely achieved.
727 Nevertheless, optimisation of radiation protection is an important task, especially because in
728 long-term missions doses to astronauts may exceed 100 mSv. The occurrence of deterministic
729 effects can also not generally be excluded. In addition, the knowledge of radiobiological
730 effects of cosmic radiation in particular heavy ions is very limited. The number of persons
731 involved is small and hence individual risk assessment is of much higher interest. As a
732 consequence, values of mean absorbed doses in organs and tissues of the human body play an
733 important role, since the weighting factors used in the definition of effective dose or
734 equivalent dose in an organ or tissue are not appropriate in the radiation field in space. In
735 addition, for many years, the use of organ dose equivalent has been preferred by many space
736 agencies instead of the quantity equivalent dose in an organ (see Chapter 3).

737 (9) Both radiation monitoring in the spacecraft environment and assessment of doses in
738 the human body of astronauts are important parts of the radiological protection measures in
739 space missions. Due to the complex radiation field and the special requirements for use in
740 space flight radiation monitoring needs specific measurement devices and procedures.
741 Usually, more than a single dosimeter type is needed for this task and often additional
742 calculations are necessary to interpret device response (see Chapters 4 and 6).

743 (10) Calculation of conversion coefficients which relate values of particle fluence or dose
744 external to the human body to values of absorbed dose and mean quality factors in organs and
745 tissues within the body is an important task and is often used for the assessment of doses in
746 the body from external measurements. While reference data on conversion coefficients
747 related to the reference voxel phantoms defined in ICRP Publication 110 have already been
748 published by the Commission in Publication 116 (ICRP, 2011), data for heavy ions have
749 become available only recently (Sato et al., 2010). In this report data are presented for
750 isotropic exposure of both male and female voxel phantoms. Omni-directional exposure
751 (ISO) is the most realistic exposure situation in space. While shielding effects may result in a
752 more non-isotropic exposure, the movement of the astronauts within the spacecraft balances
753 this situation. Hence, data are presented for ISO exposure only (see Chapter 6 and the
754 Annex). The use of conversion coefficients is, however, not the only method in assessing
755 organ doses in the body. Based on knowledge of the radiation field outside a spacecraft,
756 calculation of organ doses can be performed including the full radiation transport through the
757 walls and the equipment of a spacecraft (see Chapter 6).

758 (11) While on Earth biological dosimetry is mainly restricted to applications in accidental
759 exposure situations due to the usually low doses of occupationally exposed workers and the
760 difficulty in measuring doses below about 50 mSv by this method with acceptable
761 uncertainty, the situation in space is quite different. Mission doses may be above that
762 “threshold” and biological dosimetry (e.g. study of biological effects on lymphocytes in the
763 human body) allows a very individual assessment, if in advance the individual sensitivity is
764 determined and hence an individual calibration is performed. Methods and measurements are
765 discussed in both Chapters 4 and 6 of the report.

766



768

769

2. RADIATION ENVIRONMENT IN SPACE

770

771 (12) The radiation environment in space is a complex mixture of particles of solar and
772 galactic origin with a broad range of energies. For radiological protection, the relevant
773 radiation fields are the galactic cosmic radiation (GCR), particles ejected from the Sun during
774 solar energetic particle (SPE) events, and secondary radiation produced through interaction
775 with the planet's atmospheric nuclei. Solar wind particles, even when enhanced due to higher
776 solar activity, do not significantly contribute to the radiation exposure to man due to their
777 relative low energy and hence their absorption in already very thin shielding materials.
778 Nevertheless, the solar wind modulates the fluence rate of galactic cosmic radiation in the
779 energy range below about 1 GeV/u. During phases of higher solar activity, the cosmic
780 radiation fluence rate is decreased by a factor of three to four compared with phases during
781 minimum solar activity.

782 (13) Presently, there is no measurable contribution to the radiation exposure by primary
783 electromagnetic ionising radiation such as from solar Roentgen flares like that which
784 occurred on November 4th 2003 UTC 19:29 or from conspicuous extreme gamma radiation
785 bursts such as that which occurred on December 27th 2004 UTC 21:30:26.55; hence, they are
786 omitted, although on geological time scale, their impact on the biosphere might have been
787 significant. Secondary electromagnetic radiation of course contributes as bremsstrahlung
788 emitted from charged particles upon penetration through matter and as gamma radiation from
789 the decay of neutral pions π^0 created in the Earth's atmosphere.

790 (14) From the point of view of radiological protection, the focus is on the particulate
791 components of space radiation of ions and electrons only. Electrons might become relevant
792 during extravehicular activities (EVAs) or if manned activities in the outer radiation belts
793 become an issue which, however, will not be the case for the foreseeable future.

794 (15) Through the Earth's magnetic field and an atmospheric shield with a thickness of
795 about 1000 g cm⁻², the exposure to cosmic radiation on the Earth surface is reduced to a
796 negligible level. Leaving Earth, astronauts are shielded by the structure of the spacecraft and
797 its interior by an average of about 20 g cm⁻², a shielding close to that of the Martian
798 atmosphere, but when in low Earth orbit, they are still protected by the Earth's magnetic field
799 which limits even the exposure to solar energetic particles to a level far below the cause of
800 early radiation effects in man.

801 (16) In the absence of sporadic solar particle events, the radiation exposure in near Earth
802 orbits inside spacecraft is determined by the galactic cosmic radiation (protons and heavier
803 ions) and by the protons inside the South Atlantic Anomaly (SAA), an area where the
804 radiation belt comes closer to the Earth surface due to a displacement of the magnetic dipole
805 axes from the Earth's centre. In addition, there is an albedo source of neutrons produced as
806 interaction products of the primary galactic particles with the nuclei of the Earth atmosphere.
807 Outside the spacecraft, the exposure of astronauts is dominated by the electrons of the horns
808 of the radiation belt located at about 60° latitude in Polar regions.

809 (17) All these radiations from different sources and their interactions by various
810 mechanisms determine the actual field of ionising radiation at any given time and location
811 within the heliosphere. Its complexity is unrivalled by anything we know from terrestrial
812 experience. The radiation field inside a spacecraft is even more complex through the
813 interaction of the high energy particles with the spacecraft shielding material and the body
814 tissues of the astronauts.

815 (18) In deep space missions the Earth's radiation belts will be crossed in a couple of
816 minutes and therefore its contribution to their radiation exposure is quite small. However, the
817 subsequent protection by the Earth's magnetic field is then lost, leaving only mission
818 planning and shielding measures as a means of exposure reduction. The following sections
819 describe the radiation field in space and the interaction of the charged particles with the
820 magnetic field and shielding materials. Some numbers are given on the radiation exposure in
821 low Earth orbits and in interplanetary missions.

822

823 **2.1 Primary radiation fields**

824

825 (19) Three major primary sources of radiation can be specified in space.

- 826 • The solar system with the Sun at its centre is embedded in a complex mixture of
827 ionising radiation, galactic cosmic radiation (GCR), which continuously enters the
828 heliosphere from all directions. Inside the heliosphere, the GCR fluence rate and
829 particle energy distributions are modulated by the interplanetary magnetic field
830 produced by the charged particles continuously emitted by the Sun, the so-called solar
831 wind.
- 832 • In addition to the solar wind, the Sun occasionally emits unusually large pulses of
833 energetic particles – mostly protons and electrons with a small and variable
834 contribution from helium and heavy ions - ejected in to space by these solar eruptions.
835 The most significant of these solar-particle events (SPEs) are produced by the
836 expulsion of large amounts of material in coronal-mass ejections (CMEs).
- 837 • Celestial bodies equipped with a magnetic moment like the Earth are surrounded by
838 toroidal belts of particulate radiation which are constantly replenished by solar particles
839 and secondary particles caused by the interaction of solar and galactic particles with the
840 atmosphere. Such radiation belts constitute an important third primary exposure source.

841 (20) Figure 2.1 illustrates these three sources of ionising radiation in space, their
842 respective spatial scales and the dominant role the Sun plays in modifying its composition.
843 The highest energies measured for GCR particles (Fig. 2.1) are too large to be compatible
844 with their postulated acceleration and containment by intra-galactic magnetic fields thereby
845 giving rise to speculations about extra-galactic sources for this part and hence extending the
846 spatial scales even further. The corresponding intensities, however, are too low to contribute
847 substantially to radiation exposures.

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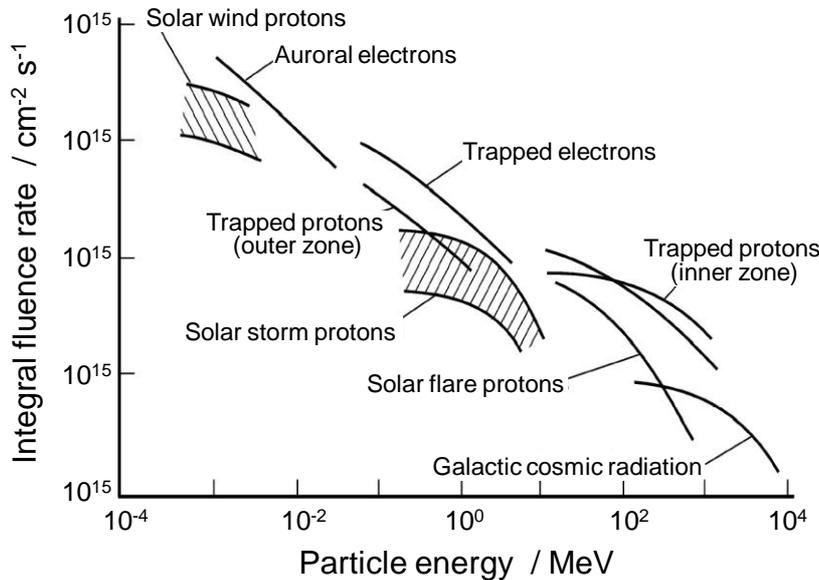


Fig. 2.1. Synoptic view of integral particle fluence rate of space radiation versus upper boundary of particle energy (Wilson, 1978).

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(21) In addition to their variation with location in space, the intensity and particulate composition in these fields are subject to temporal variations. As far as space radiation is concerned, two temporal scales of space weather events are relevant. Similar to the annual alternation between summer and winter of ordinary weather on Earth, we have to deal with a nearly regular change of solar activity between phases of maximal ('summer') and minimal ('winter') solar activity. The solar 'year' in this case is the Schwabe cycle, a period of about 11 years the duration of which however (presently) varies due to so far unknown mechanisms between 9 and 13.6 years. One measure of this activity for which a continuous observational record exists since 1755 is the Zürich sunspot number (Hathaway et al., 2002). Apparently, the maximum of solar activity is inversely associated with the length of the cycle. In addition to the field variation during the regular solar cycle, episodes of extreme solar activity characterised by explosive releases of magnetic energy (Chen, 2001) which eject giant masses of charged particles from the Sun's corona into the interplanetary magnetic field. After further acceleration in this field, particle energies up to several GeV can be attained. The impact of these solar particle events (SPE) on the radiation field in space can last for days to some weeks.

(22) Further observed solar periodicities like the magnetic Hale cycle of 22 years, the Gleisberg cycle of about 88 years or the De Vries- or Suess cycle of about 210 years have not yet been identified to substantially modulate the radiation field, although their impact on the biosphere probably is important as a recent study on glacial climate cycles discloses (Braun et al., 2005).

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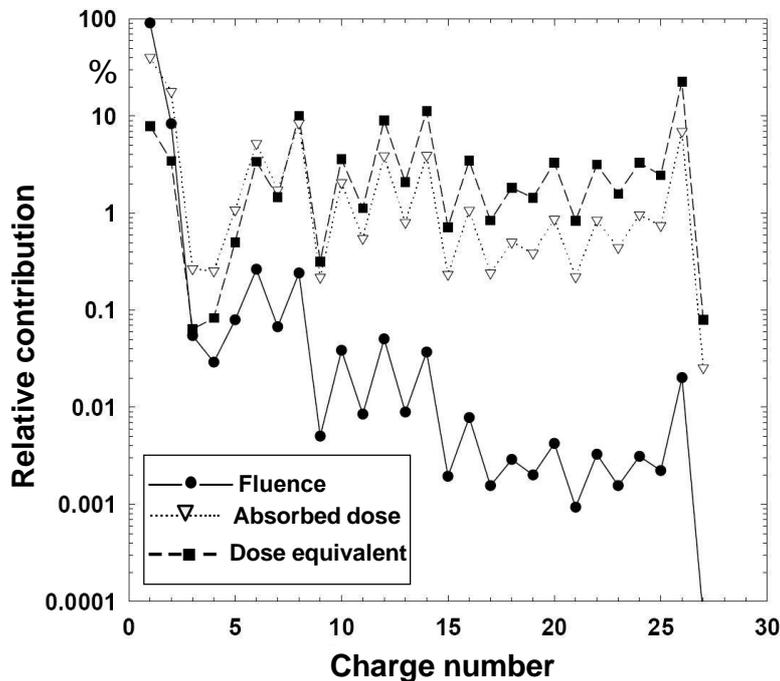
2.2 Galactic cosmic radiation

(23) Galactic cosmic radiation originates outside the solar system and impinges isotropically on Earth. There is no conclusive proof of the mechanisms accelerating the charged particles and of the astrophysical sites where matter becomes cosmic particle

880 radiation. There is no information about the directional position of their sources since these
 881 particles are scrambled by irregular interstellar magnetic fields on their way towards the
 882 Earth. Because of their high energies - up to 10^{20} eV - they most probably originate from
 883 supernova explosions, neutron stars, pulsars or other sources where high-energy phenomena
 884 are involved. Detected radiation consists of 98% baryons and 2% electrons. The baryonic
 885 component is composed of about 85% protons (hydrogen nuclei), with the remainder being
 886 alpha particles (about 14%) and heavier nuclei (about 1%). Figure 2.2 shows the abundances
 887 of these elements up to tin relative to silicon. The ions heavier than alpha particles are termed
 888 HZE-particles (high charge, with charge numbers $Z > 2$ and high energy). Although iron ions
 889 are one-tenth as abundant as carbon or oxygen, their contribution to absorbed dose in tissue is
 890 substantial, since this dose is proportional to the square of the particle charge. This is
 891 indicated in Fig. 2.2 (Cucinotta et al, 2001).

892 (24) In addition to the galactic cosmic radiation, a so-called anomalous component is
 893 observed. It consists of originally neutral particles coming from the interstellar gas which
 894 become singly ionised by solar radiation after entering the heliosphere. These particles are
 895 then accelerated in collision regions between fast and slow moving streams of the solar wind.
 896 They are able to penetrate deeper into the magnetic field than fully ionised cosmic particles.
 897 Their energies are around 20 MeV/u and consequently they can only contribute to radiation
 898 effects behind thin shielding. However, it has to be considered that they lose all their
 899 electrons after penetration of a very small amount of shielding material and thus also deposit
 900 energy proportional to the square of their charge number Z .

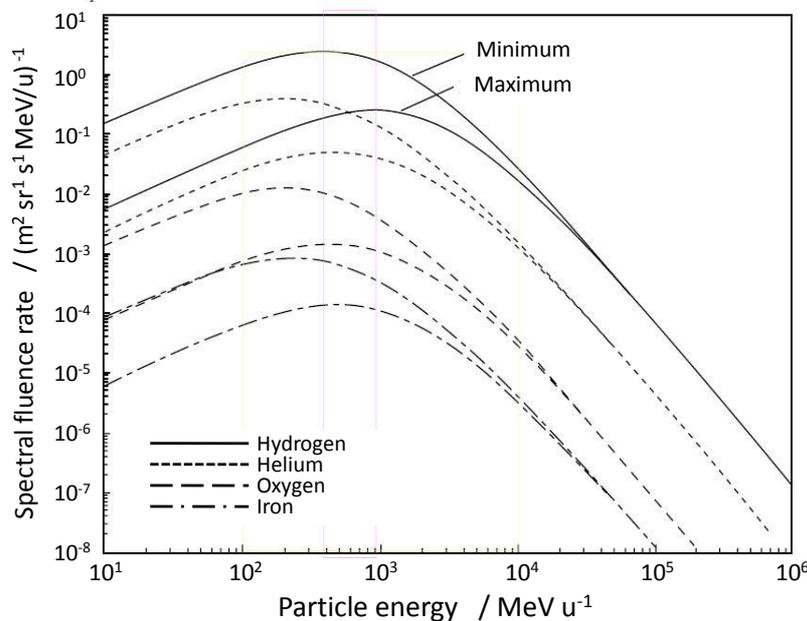
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902 Fig. 2.2. Elemental composition of galactic charged particles and their relative contribution to
 903 absorbed dose and dose equivalent in tissue (Cucinotta et al, 2001).
 904

905
 906 (25) In this report energies of GCR nuclei are always presented as kinetic energy per
 907 atomic mass unit (amu or u), E . This has the advantage that all nuclei having the same value
 908 of energy per amu move with nearly the same velocity regardless their mass. Using this
 909 energy scale the energy distributions of the different cosmic ray nuclei are very similar.

910 Fluence rate distributions in energy for hydrogen, helium, carbon and oxygen, and iron are
 911 shown in Fig. 2.3. At energies E above some GeV/u the fluence rate is well represented by a
 912 power law $N(E) \sim E^{-\gamma}$ with γ around 2.5. Towards lower energies the distributions get flatter
 913 and show a maximum at about some hundred MeV/u.

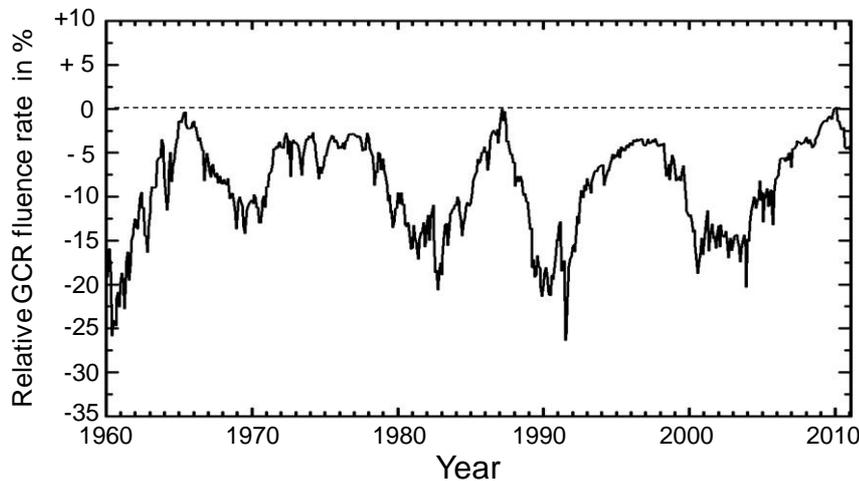


914 Fig. 2.3. Energy distribution of fluence rate versus particle energy for 4 GCR particles and their
 915 modification by solar activity (for solar minimum (1977) and solar maximum (1959)) at 1 AU as
 916 relevant for radiological protection (Badhwar, 1997).
 917

918
 919 (26) Fluence rates of cosmic radiation are not constant; they vary between two extremes
 920 which correspond in time with the maximum and minimum solar activity. Solar activity and
 921 cosmic radiation fluence rates are inversely correlated. The slope of the energy distribution in
 922 Fig. 2.3 for energies below some GeV/u is affected by this modulation of the cosmic
 923 radiation fluence rate (Badhwar, 1997). It is caused by the solar magnetic field, which is
 924 coupled to the solar wind. The solar wind is a continuous stream of highly-ionised plasma
 925 emerging from the Sun. Its intensity depends on solar activity which can be described by the
 926 number of observed sunspots. During the minimum of the 11-year solar cycle the solar wind
 927 has a minimum strength and its effect on the energy distribution is smaller than at maximum
 928 solar activity. Cosmic particles incident on the solar system interact with the solar magnetic
 929 field and thus lose energy. This leads to flattened energy spectra at lower energies. With
 930 increasing solar activity, the maximum of the fluence rate is shifted to higher particle
 931 energies. At 100 MeV/u, the particle fluence rates differ by a factor of about 10 between
 932 maximum and minimum solar activity conditions, whereas at about 4 GeV/u only a variation
 933 of about 20% is observed.

934 (27) Monitoring of solar modulation is possible on Earth based on the fluence rate of
 935 secondary neutrons produced in the Earth's atmosphere by interactions of galactic cosmic
 936 radiation. This fluence rate has been measured over longer periods by different ground based
 937 stations using neutron monitors. Figure 2.4 shows an example of data taken over several
 938 years with the neutron monitor at Kiel University (NMDB, 2011). It can be seen that details
 939 of the modulation seem to be unpredictable statistical fluctuations. However, maxima and
 940 minima clearly appear inversely correlated to the 11-year solar cycle with a roughly

941 sinusoidal form around an average particle fluence rate. However, the magnitude of the
 942 extremes again undergoes fluctuations. Predictions for future satellite missions are limited in
 943 accuracy within a factor of two or even more based on such unpredictable fluctuations.
 944



945
 946 Fig. 2.4. Relative fluence rate variation of cosmic radiation (GCR) with time in the solar cycle of
 947 the heliocentric potential measured by the neutron monitor in Kiel (NMDB, 2011).
 948

949
 950 **2.3 Solar cosmic radiation**

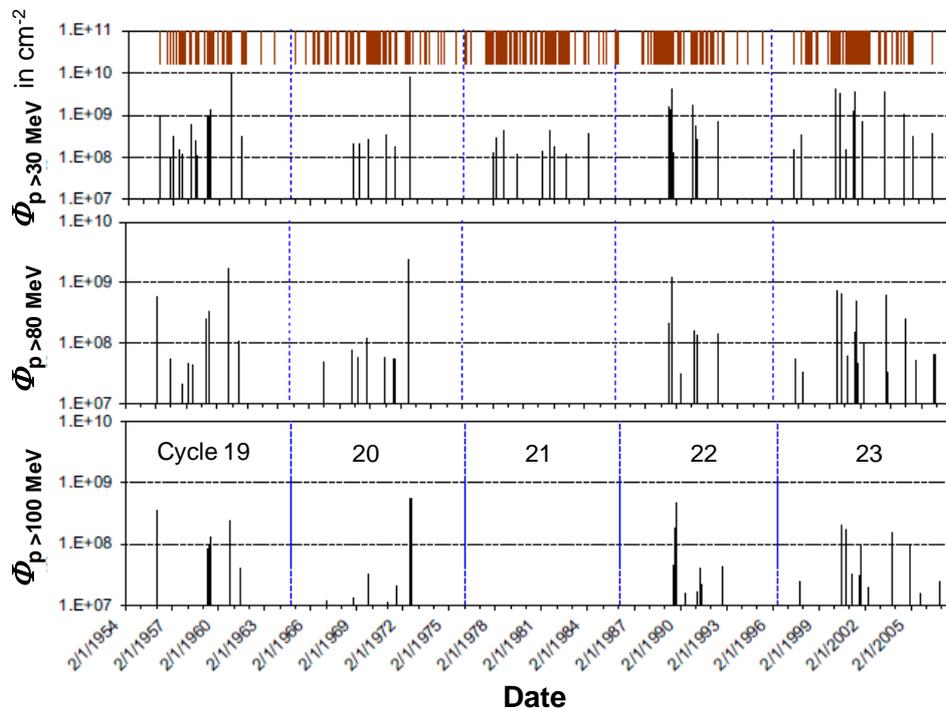
951
 952 (28) Besides electromagnetic radiation, the Sun continuously emits particle radiation,
 953 mainly consisting of protons and electrons, the solar wind. The intensities of these low
 954 energy particles vary by 2 orders of magnitude between about some 10^{10} and 10^{12} particles
 955 $\text{cm}^{-2} \text{s}^{-1} \text{sr}^{-1}$. In terms of velocity, this particle stream is characterised by velocities between
 956 about 300 km s^{-1} and 800 km s^{-1} and more. The particle energies, however, are so low (for
 957 protons between 100 eV and 3.5 keV), that the particles will be stopped within the first few
 958 microns of unshielded skin. They are, therefore, not of concern for radiation effects in man.

959 (29) Nevertheless, the temporal variation of the solar wind is a major driver which
 960 determines radiation exposure from GCR in space, at least within the inner heliosphere. The
 961 heliosphere itself can be defined as that domain of the interstellar space which the solar wind
 962 can fill out. The magnetic field based on the solar wind provides a similar shielding as the
 963 geomagnetic field does. The shielding strength can be simulated in terms of a pseudo-
 964 electrostatic heliocentric potential against which the charged particles have to work when
 965 entering the heliosphere from the local interstellar medium. This potential modifies the GCR
 966 energy spectra to the same degree as the interplanetary magnetic field does.

967 (30) Occasionally, the surface of the Sun releases large amounts of energy in sudden
 968 local outbursts of gamma radiation, hard and soft x-rays and radio waves in a wide frequency
 969 band. In these solar particle events (SPEs), large currents and moving magnetic fields in the
 970 solar corona accelerate solar matter. Coronal particles with energies up to several GeV escape
 971 into the interplanetary space. They spiral around the interplanetary magnetic field lines.
 972 Within the ecliptic plane, field lines expand from the Sun into the interplanetary medium like
 973 the stream of water from a rotating garden hose. They connect the Earth with a certain spot
 974 on the western part of the Sun. The number and energy distribution of particles observed in
 975 solar particle events at Earth is different from GCR and depends on this connection. SPEs

976 show an enormous variability in particle fluence rates and energy distribution and have the
 977 potential to expose space crew to high life-threatening doses

978 (31) An SPE well-connected with high particle fluence rates observed at Earth is an
 979 infrequent event which is most likely to be observed during the period of increasing and
 980 decreasing maximum solar activity. Therefore, major SPEs are observed at Earth as random
 981 events with a low frequency, typically one per month. They last for several hours or days.
 982 Events with significant fluence rates of protons with higher energies can be observed as
 983 "ground-level events" (GLE) by neutron monitors. Figure 2.5 shows the number of GLEs
 984 observed over the last solar cycles. Long gaps with no events can be seen during solar
 985 minimum activity. Between the last GLE in cycle 21 and the first one in cycle 22 there was a
 986 65 month quiet period which was followed by a sequence of 11 GLEs within one year with
 987 approaching maximum of the present solar cycle.
 988



989 Fig. 2.5. Occurrence of major solar particle events in solar cycles No. 19 to 23 and the integral
 990 particle fluence, Φ_p , for protons with energies >30 MeV, >60 MeV, and >100 MeV (Kim et al., 2011).
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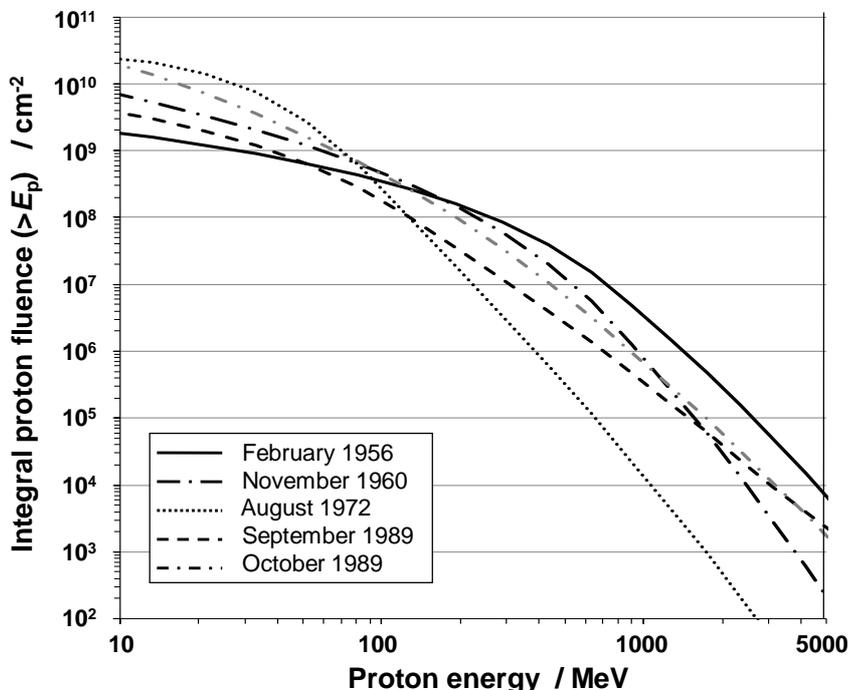
993 (32) Since high energetic particles arrive first and are followed by particles of lower
 994 energies, the energy distribution of SPE particles observed at Earth depends on time after
 995 onset of the event. Above energies of about 10 MeV, SPE particle spectra following
 996 approximately the power law $I(E) = I_0 E^{-\gamma}$. After the onset of the event, the exponent γ
 997 decreases with time. This means that the contribution by high-energy particles decreases with
 998 time during the event. The constant I_0 , describing the absolute number of particles, shows a
 999 great deal of structure during the event caused by field irregularities and shock structures in
 1000 the interplanetary medium.

1001 (33) Such events can induce adverse skin reactions in astronauts if they get caught
 1002 outside shielding since above about 10 MeV protons can penetrate space suits and reach the
 1003 skin or the lens of the eye. Depending on the particle intensities they may induce erythema or
 1004 trigger late radiation cataracts within the lens of the eye. While the latter take several years to

1005 develop and hence pose no threat to a safe mission completion, severe erythema may well
 1006 induce performance decrements which could compromise mission success.

1007 (34) Since 1955, five SPEs with intensities and energies large enough to jeopardise crew
 1008 health behind normal or even enhanced spacecraft shielding have so far been observed. For
 1009 these strong events integral fluence distributions (total number of particles per unit area
 1010 above an energy E) have been measured by satellite instruments (see Fig. 2.6). For a sixth
 1011 event - that of February 23rd 1956 - the fluence distribution has been inferred from an
 1012 analysis of the count rates of terrestrial neutron monitors which recorded the induced
 1013 secondary neutrons. Such enhancements of neutron count rates are monitored in a worldwide
 1014 net of neutron monitor stations a selected subset of which forms the so called Spaceship
 1015 Earth. Ground level enhancement (GLE) events indicate that associated SPE protons with
 1016 energies above about 450 MeV were sufficiently numerous to raise the neutron fluence rate at
 1017 sea level by at least 5%.

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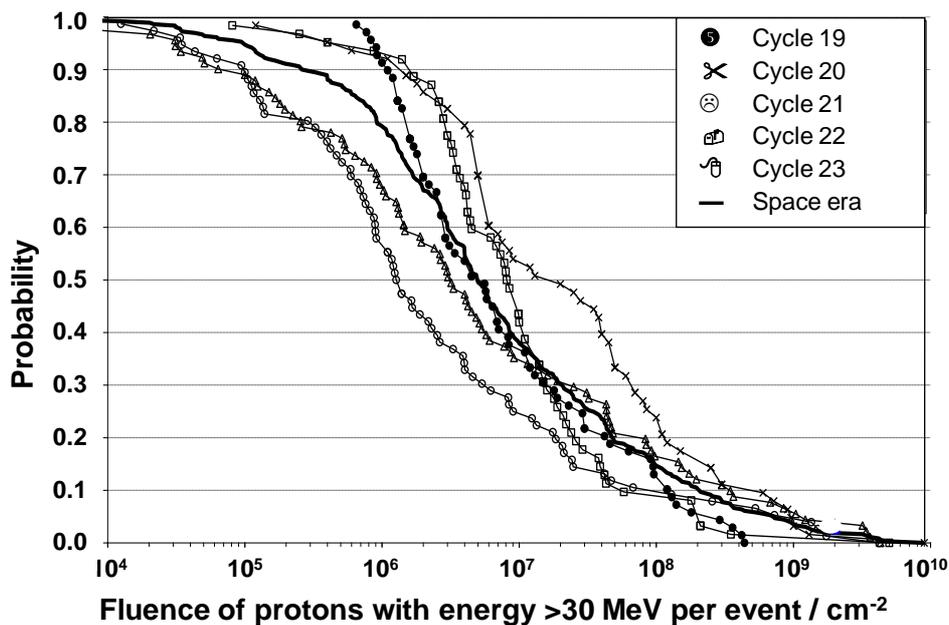
1019 Fig.2.6. Integral fluence spectra of protons from historical 'worst-case' solar particle events (Kim et
 1020 al., 2010a).
 1021

1022
 1023 (35) A comprehensive list of GLE's observed since February 23rd 1956 (GLE No. 5) and
 1024 July 14th 2000 (GLE No. 59) together with all neutron monitor stations where these events
 1025 were observed is provided by the Australian Antarctica Data Centre
 1026 (<http://data.aad.gov.au/aadc/gle/events.cfm>). Among all these GLEs, the enhancement by
 1027 GLE No. 5 in Leeds (lat.: 53.83 N, long.: 358.42 E, alt.: 100 m, Pc=2.20 GV) is about 4600%
 1028 higher than the pre-event count rate, whereas for other SPEs the enhancement very rarely
 1029 exceeds a 100% increase.

1030 (36) A small fluence rate of solar particles with low energies also reaches the Earth from
 1031 SPEs at other positions of the Sun which are not fully directed to the Earth. These fluence
 1032 rates add up to a solar component which dominates over the galactic component at energies
 1033 below 30 MeV/u. Depending on the conditions of the interplanetary medium, this
 1034 component undergoes fluctuations which are highly variable and unpredictable. During

1035 periods of maximum solar activity, when the fluence rate of galactic cosmic radiation is
 1036 depressed and SPEs are more frequent, the contribution of the solar component is more
 1037 significant.

1038 (37) For long-term mission planning, in addition to the magnitude that a worst case event
 1039 can attain, the frequency of occurrence of events as well as the proton energy spectrum
 1040 becomes important too. Fig. 2.7 gives a probability that a particle event with protons of
 1041 energies above 30 MeV occur, based on the random nature of SPE occurrence and event size
 1042 and based on the records taken for fluence measurements of the last five solar cycles
 1043



1044 Fig. 2.7. Probability of an SPE event with protons exceeding the energy of 30 MeV versus proton
 1045 fluence per event for different cycles and for the complete space era (Kim et al., 2011).
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 1049 **2.4 Trapped radiation**
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1051 (38) The radiation field around the Earth comprises the third radiation source. The
 1052 particles trapped in the radiation belts discovered by van Allen are a result of the interaction
 1053 of GCR and SCR with the Earth's magnetic field and the atmosphere. The radiation belts
 1054 consist of electrons and protons, and some heavier ions. Electrons reach energies of up to 7
 1055 MeV and protons up to 700 MeV. The energy of heavy ions is less than 50 MeV/u, and
 1056 because of their limited penetration capacity, they are of no consequence for satellite
 1057 electronics or radiological protection of humans. Charged particles with these energies
 1058 moving into a dipole field can never enter into inner areas of this field. However, if they are
 1059 put into this field for any reason, they are restricted to certain positions and cannot escape.
 1060 They move in spirals along the geomagnetic field lines and are reflected back between the
 1061 magnetic poles, acting as mirror points. Different processes contribute to fill in particles into
 1062 the radiation belt and two main zones of captured particles are observed. The inner belt is
 1063 mainly formed by decaying neutrons, coming from the atmosphere in which they are
 1064 produced in cosmic particle interactions, and producing protons and electrons. The outer belt

1065 consists mainly of trapped solar particles, and is populated largely by electrons. During
1066 disturbances of the magnetosphere by magnetic storms related to solar flares, where the
1067 geomagnetic cut-off is usually depressed, particles of lower energies can penetrate from
1068 outside towards the inner regions and fill them. The radiation belts extend over a distance
1069 from Earth from about 200 km to about 75000 km around the geomagnetic equator. Energy
1070 loss by cyclotron radiation and by penetration into the upper atmosphere near the
1071 geomagnetic mirror points constitutes the major loss mechanisms for the trapped particle
1072 population.

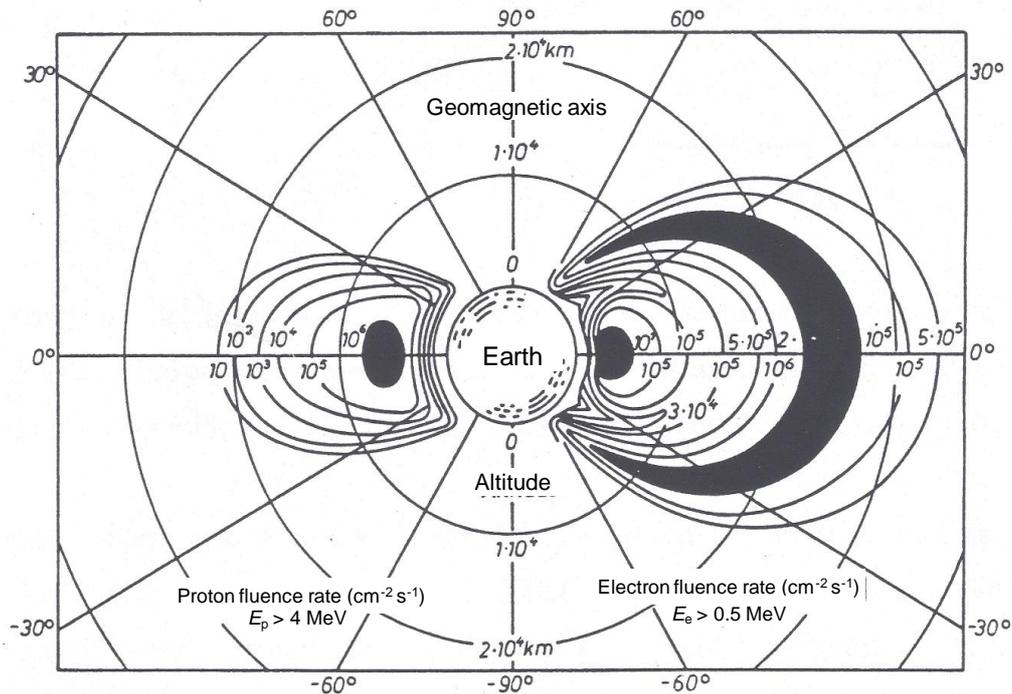
1073 (39) Extensive measurements during the recent decade with more advanced and
1074 dedicated instrumentation on several satellites in well-coordinated orbits yielded the main
1075 quantitative database which then became integrated in the AP-8 TRAPPED PROTON
1076 MODEL (Sawyer et al., 1976) and which provides energy spectra of average proton fluence
1077 rates during quiet magnetospheric conditions. A major application which these models have
1078 been designed for is the assessment of the radiation exposure from trapped radiation during
1079 manned low Earth orbit (LEO) missions such as presently on the International Space Station
1080 (ISS). The AE-8 TRAPPED ELECTRON MODEL (Vette, 1991) serves the same purpose of
1081 prediction radiation doses yet mainly for the radiation environment in geostationary orbits
1082 where energetic electrons constitute the dominant source of ionising radiation.

1083 (40) An improved AE-9/AP-9 Model is being developed as part of the Proton
1084 Spectrometer Belt Research (PSBR) Program and is planned to be released in near future by a
1085 consortium of institutions, such as the National Reconnaissance Office (NRO), Aerospace
1086 Cooperation, the Air Force Research Laboratory (AFRL), Los Alamos National Laboratory
1087 (NAL) and the Naval Research Laboratory NRL).

1088 (41) There is a strong east-west effect in trapped proton fluence rates. At the bottom of
1089 their path around the magnetic field lines, protons are travelling eastwards, whereas those on
1090 the top of their path are travelling westwards. The westwards travelling particles have
1091 emerged from a region of the atmosphere at lower altitude. Therefore, they encounter a
1092 denser atmosphere and are more efficiently removed by interactions with the nuclei of the
1093 atmosphere (Lenchek and Singer, 1962).

1094 (42) Figure 2.8 displays the spatial distribution of electron fluence rate for electron
1095 energies above 0.5 MeV (right) and of proton fluence rate for proton energies above 34 MeV
1096 (left) at which energy the latter are able to penetrate about 1.4 g cm^{-2} Al, the shielding
1097 provided by lighter space craft. Proton fluence rates in the inner belt are intense and protons
1098 reach sufficient energies to penetrate the shielding provided by walls and equipment of space
1099 craft so that primarily their energy distribution as shown in Fig. 2.9 have to be known in
1100 order to assess radiation exposures of astronauts. The data in Fig. 2.9 are the results of
1101 measurements of the energy distribution of trapped proton fluence rate in the early 1960s
1102 (Filius, 1965; Freden et al., 1964). As a natural coordinate system to specify the satellite
1103 position within the geomagnetic field, the (B, L) coordinates are used. Here, B denotes the
1104 magnetic field strength at a given point and L the altitude in units of Earth radii at which the
1105 magnetic field line through this point intersects the plane through geomagnetic equator.

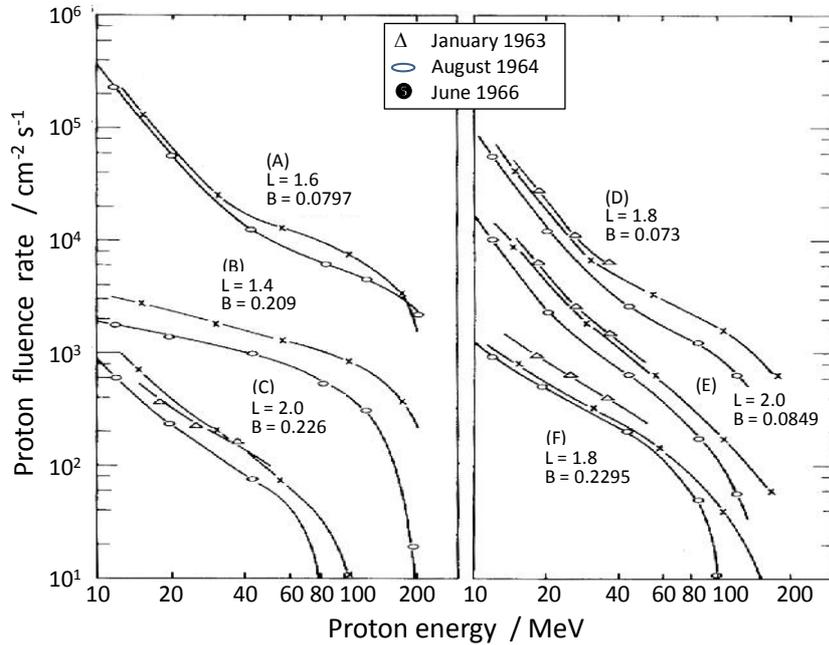
1106
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 1109 Fig. 2.8. Fluence rates in inner and outer terrestrial radiation belts for trapped protons and
 1110 electrons in $\text{cm}^{-2} \text{s}^{-1}$ (Allkofer, 1975).
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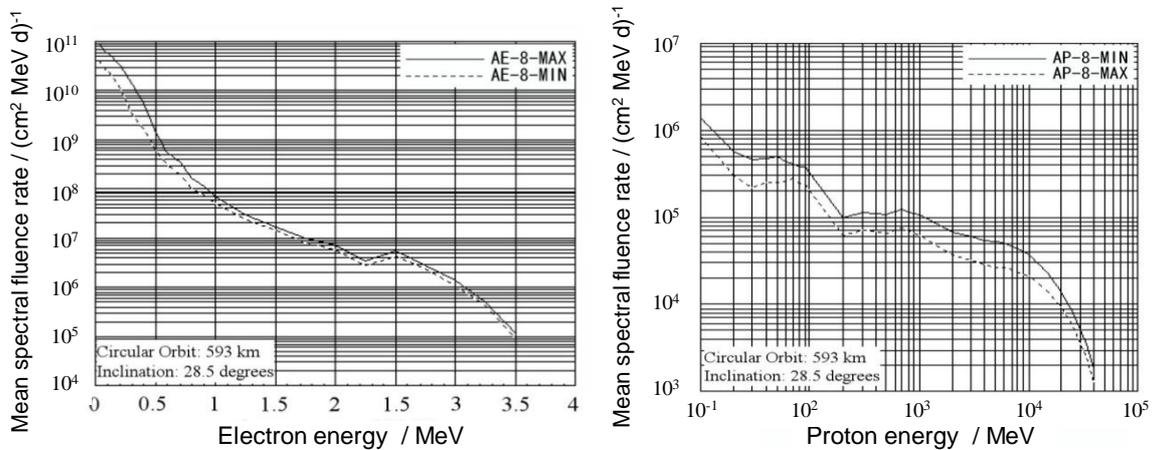
1112 (43) Figure 2.10 shows the fluence rate energy distributions of trapped electrons and
 1113 protons averaged over the orbit of the Hubble Space Telescope. Electron fluence rates during
 1114 solar maximum are greater than during solar minimum, pointing to the Sun as the dominant
 1115 primary source which feeds the trapped electron population. In contrast, the trapped proton
 1116 fluence rates reflect the (Forbush) modulation of the GCR intensity by the solar wind which
 1117 results in higher intensities during solar minimum conditions.
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Fig. 2.9. Fluence rate spectra of inner belt protons measured at various locations within the belts as expressed in the B,L -coordinate system (Filius, 1965; Freden et al., 1964).



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Fig. 2.10. Left: Mean energy distribution of fluence rate of trapped electrons at solar minimum and maximum conditions calculated with the AE-8 model for the position of the Hubble space telescope (Jones, 2000). Right: Mean energy distribution of fluence rate of trapped protons at solar minimum and maximum conditions calculated with the AP-8 model for the position of the Hubble space telescope (Jones, 2000).

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(44) The fluence rates and energy distributions shown pertain to quiet magnetic conditions of the terrestrial and interplanetary magnetic field during the minimum and maximum of solar activity. In addition to the regular solar cycle variation, both magnetic storms and intensive fluence rates from energetic solar particle events (SPE) significantly shift positions and energies of trapped particle populations so that even additional though transient radiation belts can be created.

1137 (45) The trapped radiation is modulated by the solar cycle: with increasing solar activity,
1138 proton intensity decreases, while electron intensity increases. Diurnal variations by a factor of
1139 between 6 and 16 are observed in the outer electron belt, and short term variations due to
1140 magnetic storms may raise the average fluence rate by two or three orders of magnitude. The
1141 centre of the inner belt is quite stable, especially with respect to protons. However, at the
1142 lower edge of the belt, electron and proton intensities may vary by up to a factor of 5. For the
1143 majority of space missions in low Earth orbit (LEO), protons are an important part of the
1144 radiation exposure inside spacecrafts. Because of their higher energies and correspondingly
1145 longer range, their total dose surpasses that of electrons at shielding thickness above about
1146 0.3 g/cm^2 aluminum. At lower shielding, e.g. in case of extravehicular activities (EVA), the
1147 absorbed dose to the skin is dominated by the electron contribution and may reach up to
1148 10 mGy per day.

1149 (46) Of special importance for low Earth orbits is the so called 'South Atlantic Anomaly'
1150 (SAA), at the moment a region over the coast of Brazil, where the radiation belt extends
1151 down to altitudes of 200 km. This behaviour is due to an 11° inclination of the Earth's
1152 geomagnetic dipole axis from its axis of rotation towards northern America and a 500 km
1153 displacement of the dipole centre towards the western Pacific, with corresponding
1154 significantly reduced field strength values. Radiation received in LEO at low inclinations
1155 includes GCR and radiation due to passages through the SAA. At an orbit with 28.5°
1156 inclination, six orbital rotations per day pass through the anomaly, while nine per day do not.
1157 Although traversing the anomaly takes less than about 15 min and occupies less than 10% of
1158 the time in orbit, this region accounts for a significant fraction of total exposure.

1159 (47) In addition to the trapped charged particles in radiation belts, the GCRs are producing
1160 secondary neutrons by nuclear reactions in the upper atmosphere of the Earth. Neutrons are
1161 produced in practically two energy regions by two processes. Neutrons in the region between 1
1162 and 10 MeV are mostly evaporation products of highly excited nuclei with a fairly isotropic
1163 angular distribution. High-energy neutrons originate as knock-on neutrons mainly in peripheral
1164 collisions or in charge exchange reactions of high-energy protons. Their energy distribution
1165 peaks at about 100 MeV. They leak into the exosphere and contribute also to the exposure in
1166 spacecraft. Measured neutron spectra in the Earth atmosphere are shown in Fig. 2.11. Their
1167 contribution to the radiation field in LEO is, however, relatively low. A similar neutron field as
1168 measured in the atmosphere is of course produced by interactions of the GCR with the
1169 spacecraft material and the astronaut's body. This contribution to the exposure of astronauts is
1170 substantial (Bartlett et al., 2006).

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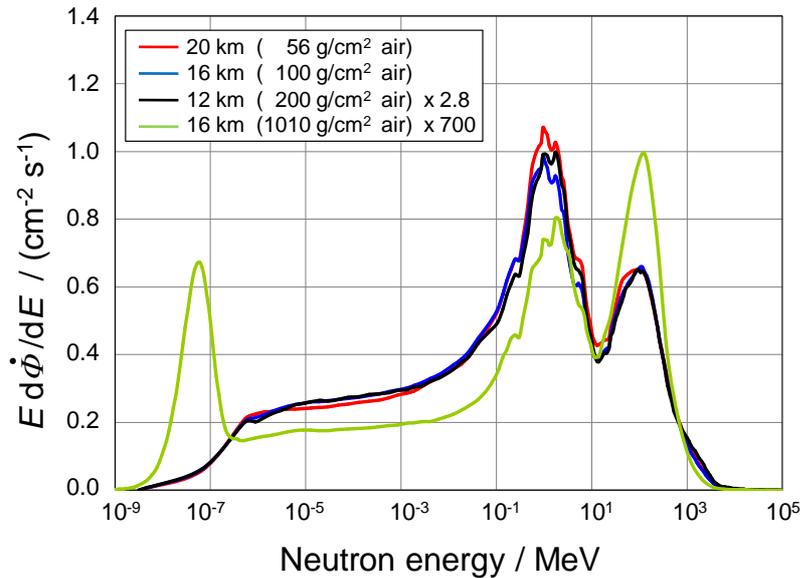


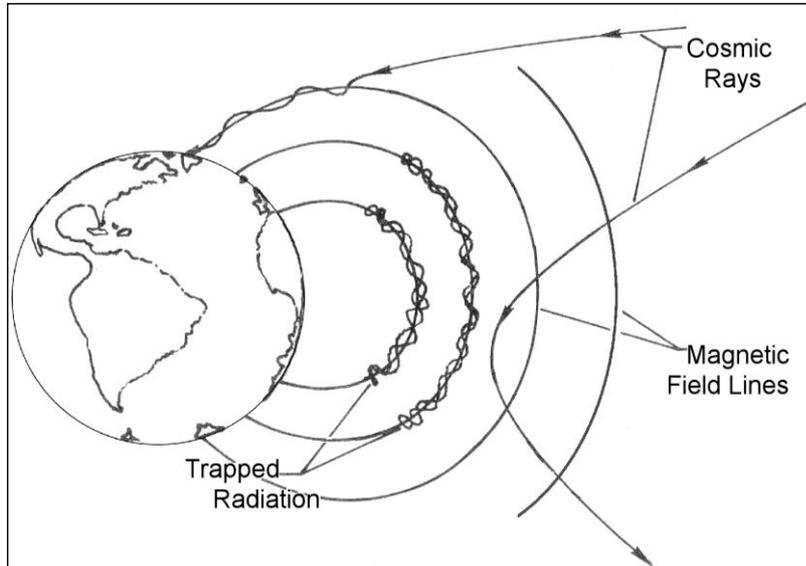
Fig. 2.11. Neutron spectra from cosmic radiation on ground and in different atmospheric depths at air flight altitudes measured with Bonner spheres during the AIR ER 2 flights and on the ground at sea level (Goldhagen et al., 2004).

2.5 Interaction with magnetic fields

(48) To reach spacecraft in low Earth orbits (LEO) a charged particle has to penetrate the Earth's magnetic field. Penetrability is a property related to the ions magnetic rigidity which is given by its momentum divided by its charge. All particles with the same rigidity follow a track with the same curvature in a given magnetic field. For each point inside the magnetosphere and each direction from that point, there exist a rigidity threshold below which the cosmic particles are not able to reach this point. This rigidity is called the geomagnetic cut-off rigidity and is proportional to the magnetic field component perpendicular to the direction of particle motion. For a particle moving towards the centre of Earth, for example, the cut-off rigidity has a maximum value at the equator, since the particle moves perpendicular to the field lines and the cut-off rigidity vanishes at the pole, since the particle moves in the direction of the field lines. Therefore, geomagnetic shielding is less effective for high inclination orbits than for low inclination orbits. This means that in low inclination orbits, only particles of high energy have access. Towards higher inclinations, additional particles of lower energies are observed. For a geomagnetic latitude λ , the vertical cut-off rigidity R_c can be calculated approximately by $R_c = 14.9 \cos^4 \lambda / (r/r_e)^2$, where r/r_e is the ratio of the distance r from the dipole centre to the Earth radius, r_e . The rigidity for particles arriving from other directions than vertical is dependent from the angle of incidence. Due to latitude dependent shielding, the number of particles incident in the altitude of orbiting spacecraft increases from lower inclinations towards higher inclinations.

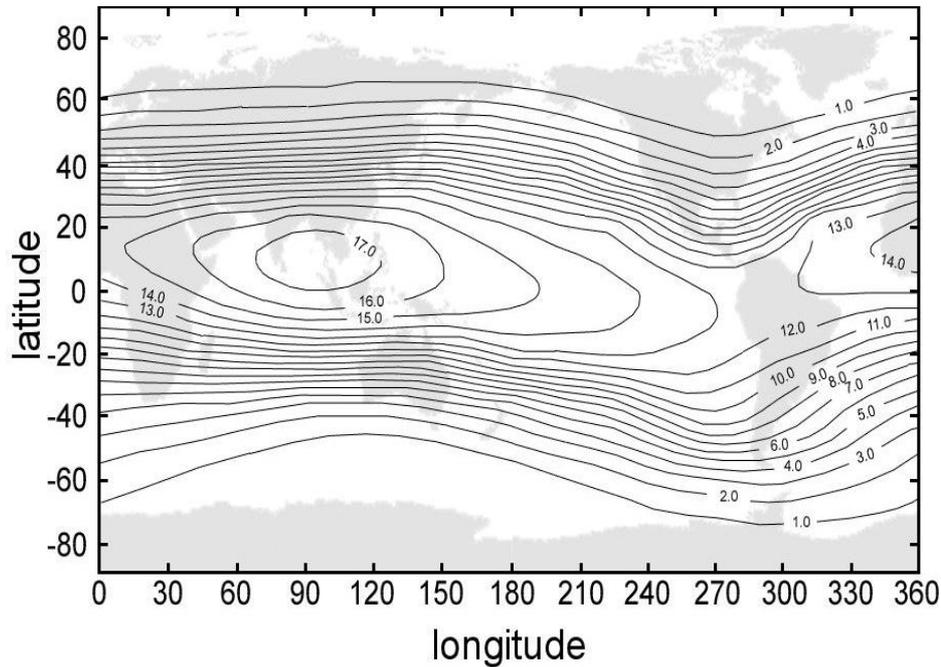
(49) In LEO, as well as in aviation heights, a second shielding mechanism has to be incorporated into the transport of the primary GCR or SPE ions. Whereas the geomagnetic field on the one hand is responsible for the added radiation exposure in LEO from trapped radiation, it also causes a quite substantial reduction of radiation exposure, at least near the geomagnetic equator (which differs from the geographic equator). This stems from the

1205 deflection due to the Lorentz force of charged particles by the geomagnetic field as illustrated
 1206 in Fig. 2.12.
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 1209 Fig. 2.12. Deflection and trapping of charged particles by the geomagnetic field (Spjeldvik et al.,
 1210 1983).
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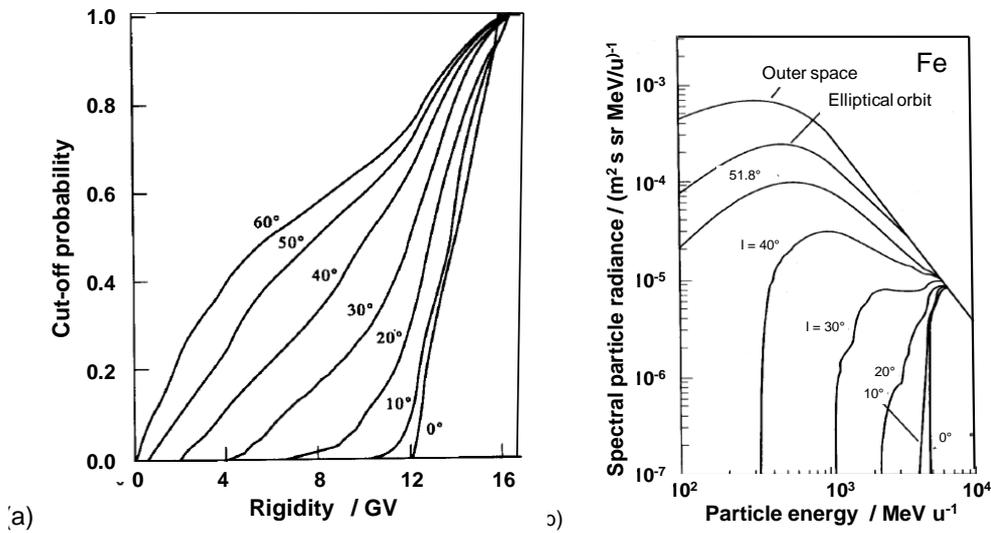
1212 (50) Figure 2.13 provides a global map of the vertical cut-off rigidities for the
 1213 geomagnetic field model of Epoch 2000 (Smart et al., 2008). For a homogeneous dipole field,
 1214 the iso-rigidity lines would be parallel to the (geomagnetic) equator. The marked asymmetry
 1215 with a peak above 17 GV of the cut-off rigidity at the Indian Ocean (long. 90E, lat: 10N)
 1216 reflects the offset from the geographic centre of the magnetic centre by about 450 km in this
 1217 direction. At the opposite side, in the South Atlantic this offset results in the corresponding
 1218 subsidence of the lower fringes of the inner proton belt creating thereby the so called South
 1219 Atlantic Anomaly (SAA). This is the reason for the already mentioned fact that the bulk of
 1220 radiation exposure in most LEOs is accumulated in this region.
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 1222



1223 Fig. 2.13. Map of vertical cut-off rigidities in GV for the geomagnetic field model of Epoch 2000
 1224 (Smart et al., 2008).
 1225
 1226

1227 (51) For a given orbit, the shielding due to this effect is expressed by the geomagnetic
 1228 transmission factor which specifies the fraction of the GCR or solar particle fluence rate of a
 1229 given particle energy (or momentum given in units of GeVc^{-1}) which has access to this orbit
 1230 or by the cut-off rigidity probability which specifies the probability that a particle with a
 1231 given rigidity reaches that orbit. Figure 2.14 (a) demonstrates the dependence of the
 1232 geomagnetic shielding on the orbit inclination for a circular orbit at 223 km altitude. For an
 1233 orbit of 28.5 degrees inclination which for a large fraction evades the SAA, on the average
 1234 the GCR with a momentum below about 4.2 GeV c^{-1} do not reach that flight route.
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Fig. 2.14. (a) Cut-off rigidity probability for different inclination for a circular orbit of 223 km (Heinrich et al., 1979).

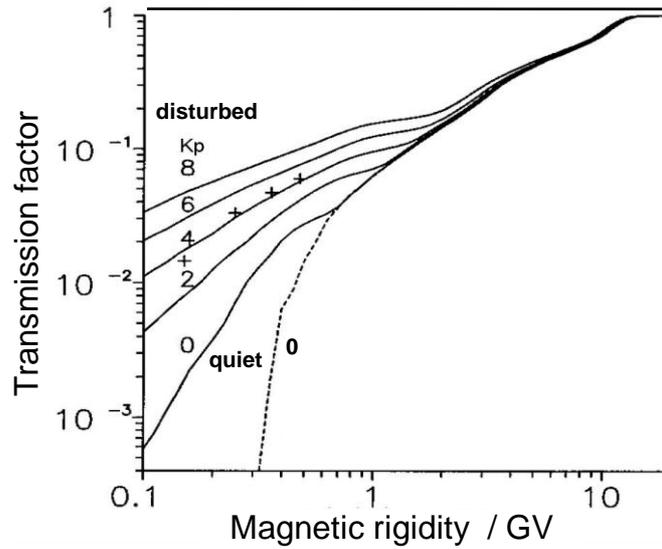
(b) Energy spectra of iron nuclei for outer space and for orbits in 223 km altitude of different inclinations (Heinrich et al., 1979).

1243

1244 (52) For a 45 degree inclination, this momentum threshold drops to about 1.1 GeV c^{-1} ,
1245 whereas for polar orbits at least 20% of particles with the lowest energies always have access
1246 to this altitude. On the other hand the shielding effect vanishes for ions with a momentum
1247 above about 15 GeV c^{-1} , where at any inclination all charged ions reach this orbit. Figure 14
1248 (b) shows the influence of the magnetic shielding on the particle spectra, e.g. for Fe, which
1249 varies strongly with the different inclination. The functions in Fig. 2.14 do not, however,
1250 include the shadow effect of the Earth itself.

1251 (53) The shadow effect of the Earth for the Hubble Space Telescope (HST) at 28.5
1252 degree inclination reduces the fluence rate of even the most energetic GCR by about 30%. An
1253 Earth observation satellite such as e.g. TERRA, on the other hand, must use a near polar orbit
1254 and therefore can be accessed by charged particles of all energies. Its higher altitude also
1255 slightly reduces the shielding by the Earth shadow. The high inclination of the International
1256 Space Station (ISS) of 51.6 degree makes this manned spacecraft accessible to SPE ions of
1257 100 MeV/u or above. This is particularly important, since in case of geomagnetic
1258 disturbances which often accompany solar events, this geomagnetic shielding is further
1259 reduced. Fig. 2.15 demonstrates this loss of geomagnetic shielding for the ISS for storms as
1260 characterized by the K_p index of global geomagnetic activity (see:
1261 http://isgi.cetp.ipsl.fr/des_kp_ind.html) which can vary between 0 and 9. Under conditions of
1262 such storms a much larger fraction of SPE ions can reach the orbit of the ISS.

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Fig. 2.15. Calculated geomagnetic transmission functions for the ISS orbit versus magnetic rigidity for a quiet time geomagnetic field (dotted line) (IAGA, 1992) and for disturbed geomagnetic conditions (solid lines) as expressed by the K_p index (Nymmik, 1999). The crosses are the results from Boberg et al. (1993) for $K_p = 5$.

1270 3. QUANTITIES USED IN RADIOLOGICAL PROTECTION

1271

1272 (54) The description and quantification of exposure of humans to ionising radiation needs
1273 the definition of specific quantities and units. For many years this has been performed by
1274 ICRU and ICRP and the actual definitions can be found in their publications (ICRU, 1993;
1275 2011 and ICRP, 2007). Human exposure to ionising radiation can occur from radiation
1276 incident on the human body (external exposure) or from radionuclides incorporated in the
1277 body (internal exposure). For exposure of astronauts in space, internal exposure is of very
1278 little relevance and therefore this chapter concentrates on the concept of quantities relevant
1279 for external radiation exposure.

1280 (55) In the past, the definition of the specific quantities for radiological protection has
1281 mainly considered occupational and public exposure situations on Earth concentrating on
1282 human exposures by photons, electrons and neutrons, and also α -particles in internal
1283 exposure. In addition, the protection quantities are defined for application in situations of low
1284 dose and low dose rates. Limits are given in terms of these quantities in order to limit the
1285 probability of the occurrence of stochastic detriments to humans to an acceptable level and to
1286 avoid deterministic effects (tissue reactions).

1287 (56) The exposure situation in space is different from that on Earth mainly due to the
1288 strong differences in the radiation fields (see Chapter 2) and to the higher environmental dose
1289 rate in space than on Earth whereby the likelihood of stochastic effects are increased and
1290 deterministic effects cannot be excluded. The high contribution of heavy ions in GCR and
1291 secondary radiation to doses in the human body needs to be considered in particular.

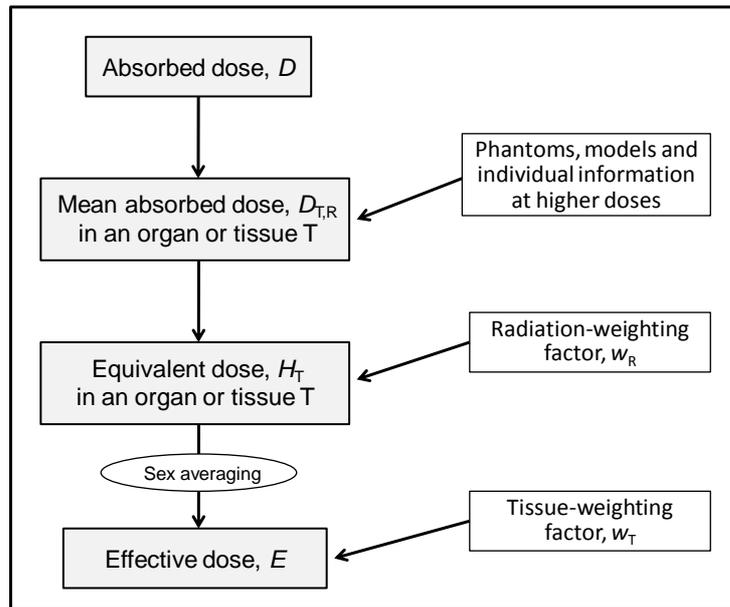
1292 (57) The consequences for the definition of appropriate radiation protection quantities for
1293 use in space situations are considered in the following sections. Organizations active in space
1294 missions have defined detailed procedures for the assessment of human exposure in space
1295 since long time (see e.g. NCRP 2000; 2002). In principle, they are interested not only in the
1296 determination of individual doses, but in the assessment of individual radiation risks based
1297 on such doses and this has some consequences for the definition of dose quantities. The
1298 astronauts can be treated as single persons, or as members of small groups. Information on
1299 sex and age can be applied and better information on the astronaut's organ size and shape
1300 might be available, and there may be the possibility of applying individual sensitivity factors
1301 to the assessed organ doses if specific individual risks need to be determined (Atwell, 1994;
1302 Bahadori et al., 2011; Bahadori et al., 2012).

1303

1304 3.1 Dose quantities in radiological protection

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1306 (58) In radiological protection, the system of dose quantities defined by ICRP and ICRU
1307 (ICRP, 1991; ICRP, 2007; ICRU, 1993) and generally used in situations on Earth and in
1308 usual aviation altitudes includes protection quantities and operational quantities. Protection
1309 quantities (e.g. equivalent dose in an organ or tissue, effective dose) are based on mean
1310 absorbed doses to the organs and tissues of the human body and can be related to the risks of
1311 ionising radiation exposure. The ICRP system of protection quantities is shown in Fig. 3.1
1312 (ICRP, 2007).



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1314

1315 Fig. 3.1. Dose quantities for radiological protection (protection quantities) recommended by the
1316 Commission (ICRP, 2007).

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1318 (59) Operational quantities (e.g. ambient dose equivalent or personal dose equivalent) are
1319 defined for use in measurements which often enable an assessment of effective dose or mean
1320 doses in organs or tissues of humans which are generally not measurable.

1321 (60) In radiation biology, clinical radiology, and radiological protection the absorbed
1322 dose, D , is the fundamental physical dose quantity. It is used for all types of ionising radiation
1323 and any irradiation geometry.

1324 (61) Absorbed dose, D , is defined as the quotient of $d\bar{\epsilon}$ by dm , where $d\bar{\epsilon}$ is the mean
1325 energy, imparted by ionising radiation to matter of mass dm , thus

1326

$$D = \frac{d\bar{\epsilon}}{dm} \quad (3.1)$$

1327 The SI unit is J kg^{-1} and its special name is gray (Gy). Absorbed dose takes account of the
1328 radiation field inside and outside the specified volume of mass dm and hence of all charged
1329 particles which were produced in or enter that volume. Its value is derived from the mean
1330 value of the stochastic quantity of energy imparted, ϵ , and does not reflect the random
1331 fluctuations of the interaction events in tissue. Generally, absorbed dose is a measurable
1332 quantity and primary standards exist to allow its determination by measurement.

1333

1334 3.1.1 Protection quantities

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1336 (62) Generally, radiological protection is concerned with controlling exposures to
1337 ionising radiation so that tissue reactions are prevented and the detriment from stochastic
1338 effects is limited to accepted levels. The fundamental *protection quantities* are based on
1339 absorbed doses imparted to organs and tissues of the human body. These quantities are
1340 designed to allow quantification of the exposure to ionising radiation from both whole and
1341 partial body irradiation (ICRP, 1991; 2007). The estimated doses can then be compared with

1342 recommended dose limits for e.g. occupationally exposed people or at higher doses mean
 1343 absorbed doses in organs and tissues may be used for the assessment of risk of cancer or
 1344 other detriments.

1345 (63) **Mean absorbed dose in an organ or tissue** In radiological protection, the main
 1346 interest is not directed to the absorbed dose at a point in the human body, but to the absorbed
 1347 dose averaged over a tissue or organ volume. The mean absorbed dose, $D_{T,R}$, in an organ or
 1348 tissue T due to a radiation of type R is the basic quantity for the definition of the protection
 1349 quantities equivalent dose and effective dose used for describing exposures at low doses and
 1350 dose rates where stochastic effects are dominant. In a mixed radiation field the mean
 1351 absorbed dose, D_T , in an organ or tissue T is given by

1352
$$D_T = \sum_R D_{T,R} \quad . \quad (3.2)$$

1353 (64) The mean absorbed dose is not always representative of the local absorbed dose
 1354 throughout an organ or tissue, e. g. for low-penetrating radiation. For strongly-penetrating
 1355 radiation, however, the absorbed dose distribution within most organs may be sufficiently
 1356 homogeneous and thus the mean absorbed dose is mostly a suitable measure of the dose
 1357 throughout the organ or tissue.

1358 (65) **Equivalent dose in an organ or tissue** The protection quantity *equivalent dose in*
 1359 *an organ or tissue*, H_T , is defined by

1360
$$H_T = \sum_R w_R D_{T,R} \quad (3.3)$$

1361 where w_R is the radiation weighting factor for radiation R which considers the differences in
 1362 the radiobiological effectiveness of different radiations (details see Section 3.2.2) and $D_{T,R}$
 1363 the average absorbed dose in the volume of a specified organ or tissue, T, due to the radiation
 1364 of type R. The radiation R is given by the type and in the case of neutrons the energy of
 1365 radiation either incident on the body or emitted by radionuclides residing within the body.
 1366 The sum is performed over all types of radiations involved. The unit of equivalent dose is J
 1367 kg^{-1} and has the special name sievert (Sv). Equivalent doses, H_T^M and H_T^F , are specified in
 1368 male and female bodies represented by male and female reference voxel phantoms.

1369 (66) **Effective dose** The effective dose, E , introduced in Publication 60 (ICRP, 1991) and
 1370 again defined in Publication 103 (ICRP, 2007) as:

1371
$$E = \sum_T w_T H_T \quad (3.4)$$

1372 where w_T is the tissue weighting factor for organ or tissue T representing the relative
 1373 contribution of that organ or tissue to the total health detriment resulting from uniform
 1374 irradiation of the body at low doses and dose rates. It is $\sum w_T = 1$. The sum is performed over
 1375 14 organs and tissues of the human body individually considered in the definition of E and
 1376 for which specific w_T values are given in Table 3.1 and an additional remainder tissue
 1377 representing a mean value of further tissues (14 tissues, but only 13 tissues for each gender)
 1378 (see Table 3.1).

1379 (67) The unit of effective dose is J kg^{-1} with the special name sievert (Sv). The same unit
 1380 is also used for the operational dose quantities. Care must be taken in ensuring that the
 1381 quantities being used are clearly stated.

1382 (68) In the calculation of effective dose the equivalent doses H_T are taken as the mean
 1383 values averaged over the male and female organs and tissues:

1384
$$H_T = 0.5(H_T^M + H_T^F). \tag{3.5}$$

1385 (69) The concept of tissue weighting factors has been introduced already in 1977 (ICRP,
 1386 1977), extended in 1991 (ICRP, 1991) and further modified in 2007 (ICRP, 2007). The
 1387 values of w_T for the specified tissues and organs are based on the detriment due to stochastic
 1388 effects after radiation exposure and on judgments. They represent mean values for humans
 1389 averaged over all ages and both sexes.
 1390

1391 Table 3.1. Tissue weighting factors, w_T (ICRP, 2007)

Organ/Tissue	w_T	Total Contribution
Lung, Stomach, Colon, Bone marrow, Breast, Remainder	0.12	0.72
Gonads	0.08	0.08
Thyroid, Oesophagus, Bladder, Liver	0.04	0.16
Bone surface, Skin, Brain, Salivary glands	0.01	0.04

1392 Notes:

- 1393 1. The w_T for gonads is applied to the mean of the doses to testes and ovaries.
 1394 2. The dose to the colon is taken to be the mass-weighted mean of ULI and LLI doses, as in the Publication 60 formulation.
 1395 The specified remainder tissues (14 in total, 13 in each sex) are: Adrenals, Extrathoracic tissue (ET), Gall bladder,
 1396 Heart, Kidneys, Lymphatic nodes, Muscle, Oral mucosa, Pancreas, Prostate(♂), Small intestine (SI), Spleen, Thymus,
 1397 Uterus/cervix (♀).
 1398

1399 (70) The use of effective dose allows exposures in very different situations (e.g. internal
 1400 and external exposure by different types of radiation and inhomogeneous exposures of the
 1401 body) to be combined and results in a single dose value which considerably simplifies the
 1402 specification of exposure limits. Effective dose, however, is not designed as a quantity
 1403 considering individual properties of a specific person and should, therefore, not be applied for
 1404 an assessment of radiations risks of a single person.
 1405

1406 **3.1.2 Operational dose quantities**

1407
 1408 (71) The protection quantities defined by mean organ or tissue doses in the human body
 1409 are not measurable in practice and, therefore, cannot be used as quantities in radiation
 1410 monitoring. For radiation measurements in situations of external exposure (area or individual
 1411 monitoring) specific operational dose quantities have been defined by ICRU (ICRU 1985;
 1412 1988; 1993; 2001). In monitoring at low doses the values of these quantities are taken as
 1413 sufficiently accurate assessments of effective dose or skin dose, respectively, if their values
 1414 are below the recommended limits for occupational exposure.

1415 (72) The basis for the definition of the operational quantities is the quantity dose
 1416 equivalent, H , defined by

1417
$$H = Q D \tag{3.6}$$

1418 where D is the absorbed dose at the point of interest in tissue and Q the corresponding mean
 1419 quality factor due to the charged particles at that point.

1420 (73) For the different tasks of monitoring of external exposures the following scheme can
 1421 be used for describing the application of the different operational dose quantities (ICRP,
 1422 2007)

1423
1424

Task	Operational dose quantities for	
	area monitoring	individual monitoring
Control of effective dose	ambient dose equivalent, $H^*(10)$	personal dose equivalent, $H_p(10)$
Control of doses to the skin, the hands and feet	directional dose equivalent, $H'(0.07, \Omega)$	personal dose equivalent, $H_p(0.07)$
Control of doses to the lens of the eye	directional dose equivalent, $H'(3, \Omega)$	personal dose equivalent, $H_p(3)$

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(74) The operational quantities for area monitoring at a location in a radiation field are defined by the dose equivalent at a point in a simple phantom, the ICRU sphere. It is a sphere of tissue-equivalent material (30 cm in diameter, ICRU (soft) tissue with density: 1 g cm^{-3} , and mass composition: 76.2 % oxygen, 11.1 % carbon, 10.1 % hydrogen and 2.6 % nitrogen). These quantities are defined to be point quantities fulfilling the condition of being additive with respect to the different components of the radiation field at the point of interest. For its definition the terms “*expanded*” and “*aligned*” radiation field are introduced (see Glossary) and the value of the quantity is given by the dose in a point at a fixed depth in the sphere. The sphere is hypothetical, a mathematical construct for the determination of the values of the quantities for area monitoring.

Ambient dose equivalent, $H^*(10)$

1437
1438
1439 (75) For area monitoring the operational quantity for assessing effective dose is the ambient dose equivalent, $H^*(10)$, defined by (ICRU 2001):

1440 The *ambient dose equivalent*, $H^*(10)$, at a point in a radiation field, is the dose equivalent
1441 that would be produced by the corresponding expanded and aligned field in the ICRU
1442 sphere at a depth of 10 mm on the radius vector opposing the direction of the aligned field.

1443 (76) In most practical situations on Earth ambient dose equivalent provides a
1444 conservative estimate of effective dose a person would receive at that position. This,
1445 however, may not be the case for persons in high energy radiation fields, e.g. in cosmic
1446 radiation fields (Pelliccioni, 1998).

1447
1448

Directional dose equivalent, $H'(d, \Omega)$

1449 (77) For area monitoring the quantity for assessing the dose to the skin and the
1450 extremities (hands, arms, feet) and the dose to the lens of the eye, the operational quantity is
1451 the directional dose equivalent, $H'(d, \Omega)$ defined by:

1452 The *directional dose equivalent*, $H'(d, \Omega)$, at a point in a radiation field, is the dose
1453 equivalent that would be produced by the corresponding expanded field in the ICRU
1454 sphere at a depth, d , on a radius in a specified direction, Ω .

1455 (78) For assessing the dose to the skin and the extremities $d = 0.07 \text{ mm}$ is used and
1456 $H'(d, \Omega)$ is then written $H'(0.07, \Omega)$. In case of monitoring the dose to the lens of the eye the
1457 quantity $H'(3, \Omega)$ with $d = 3 \text{ mm}$ was recommended by ICRU.

1458

1459 **Personal dose equivalent, $H_p(d)$**

1460 (79) Individual monitoring of external exposure is usually performed with personal
1461 dosimeters worn on the body and the operational quantity defined for this application takes
1462 this situation into account. For individual monitoring the operational quantity is the personal
1463 dose equivalent, $H_p(d)$.

1464 The *personal dose equivalent*, $H_p(d)$, is the dose equivalent in ICRU (soft) tissue (see
1465 Section 4.3) at an appropriate depth, d , below a specified point on the human body.

1466 (80) The specified point is usually given by the position where the personal dosimeter is
1467 worn. For the assessment of effective dose a depth $d = 10$ mm and for assessing equivalent
1468 dose to the skin and to the hands and feet a depth $d = 0.07$ mm is recommended. In special
1469 cases of monitoring the dose to the lens of the eye a depth $d = 3$ mm has been proposed to be
1470 appropriate.

1471

1472 **3.2 Description of radiation quality**

1473

1474 (81) The biological effectiveness of ionising radiation with respect to the induction of
1475 cancer or other tissue reactions in the human body and of hereditary effects depends on the
1476 type and energy of the radiation, on the tissue type exposed, on the dose and dose rate applied
1477 to the tissue and on the detriment considered. The induced effects differ also in the latency
1478 time between the exposure and the occurrence of the effects.

1479 (82) In radiological protection the effectiveness at low doses and dose rates are mainly of
1480 interest. For these doses and dose rates the stochastic effects - cancer induction and hereditary
1481 effects - are important and the standard approach in radiological protection is that the shape
1482 of the dose-response relationship at low doses and dose rates is based on the model that the
1483 probability of a stochastic effect is proportional to the applied dose (linear-non-threshold
1484 model (LNT)) (ICRP, 2005).

1485 (83) At higher doses, however, when non-stochastic effects (deterministic effects, tissue
1486 reactions) may occur, LNT is not an acceptable approximation. Tissue reactions occur always
1487 above a dose threshold. The threshold dose value depends on the type of the tissue reaction
1488 and is mostly above an applied absorbed dose of 0.5 - 2 Gy.

1489 (84) In principle, for risk estimates from exposure by different types of ionising radiation,
1490 the absolute values of biological effectiveness of radiation with respect to the induced effect
1491 need to be known. The mostly used procedure of taking care of the differences in the
1492 biological effectiveness, however, is to use risk factors for a reference radiation (usually
1493 photons) and to consider the differences of other types of radiation by applying a factor
1494 *relative biological effectiveness* (RBE) which has already been introduced by Failla and
1495 Henshaw in 1931 (Failla et al., 1931) and is discussed in detail in ICRP Publication 92
1496 (ICRP, 2003). While RBE-values always depend on the biological endpoint considered and
1497 on dose and dose rate applied, for the definition of quantities in radiological protection a
1498 single set of radiation weighting factors and radiation quality factors has been chosen (ICRP,
1499 2007) based on data at low doses and dose rates. This is further discussed in the next sections.

1500

1501 **3.2.1 Relative biological effectiveness**

1502

1503 (85) In radiobiology the differences in the effectiveness of the different ionising
1504 radiations have led to the definition of the *relative biological effectiveness* (RBE). RBE
1505 values are given as the ratio of the absorbed doses of two types of radiation producing the

1506 same specified biological effect under identical irradiation conditions (dose value of a
1507 reference radiation divided by the corresponding dose value of the considered radiation at the
1508 same level of the specified biological effect). RBE values depend on the conditions of
1509 exposure including the biological effect investigated, the tissue or cell type exposed, the dose
1510 and dose rate, and the dose fractionation scheme. Therefore, for a given type of radiation,
1511 experimental investigations often provide a large range of RBE values (ICRP, 2003). As the
1512 basis for selecting radiation weighting factors (see 3.2.2), RBE values with respect to
1513 stochastic effects (cancer induction and hereditary effects) are of main interest. Usually RBE-
1514 values increases with decreasing dose and dose rate and are believed to reach maximum
1515 values (RBE_M) at low doses and low dose rates. RBE_M -values are most relevant for
1516 radiological protection applications. In ICRP Publication 92 (2003), different methods are
1517 discussed to determine RBE_M at low doses and dose rates. The discussion includes various
1518 types of high-LET radiation, e.g. protons, neutrons and α -particles. For heavy ions, however,
1519 very limited data were available at that time.

1520 (86) Generally, low-LET radiation is taken as reference radiation, mostly high-energy x-
1521 rays or γ -rays from ^{60}Co or ^{137}Cs . There are, however, substantial differences in the biological
1522 effectiveness of photons of different energy (Schmidt et al., 2002). Nevertheless, there exists
1523 no international recommendation on defining a specific photon source as a general reference
1524 radiation. This has the consequence that a broad range of experimental data with photons has
1525 been used as a basis for the definition of radiation weighting and quality factors (ICRP,
1526 2003).

1527 (87) While on the one hand RBE_M -values are the basis for the definition of radiation
1528 weighting factors and the $Q(L)$ -function, on the other hand, RBE-values are used for the
1529 assessment of radiation risks from human exposures by different types end energies of
1530 radiation. A risk factor for the radiation and tissue of interest is obtained by multiplying the
1531 risk factor for the reference radiation, e.g. photons, by the corresponding RBE-value. An
1532 uncertainty for risk assessment, however, comes in, because RBE-values are often
1533 determined by investigating biological effects in single cells or small animals at higher doses,
1534 while radiation risks are assessed for stochastic effects, e.g. cancer induction and hereditary
1535 effects, in humans.

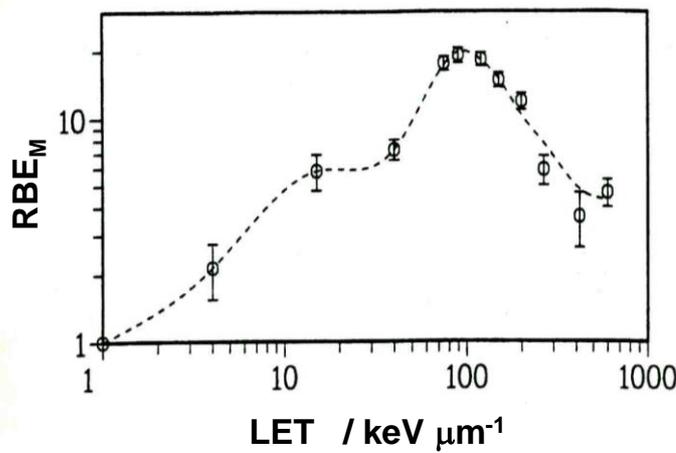
1536 (88) The biological effectiveness of ionising radiation is known to be correlated with its
1537 energy deposition properties along the tracks of charged particles, especially with the
1538 ionisation density along their tracks. For applications in radiological protection, the complex
1539 structure of the charged particle tracks in tissue is characterized by a single parameter only,
1540 the unrestricted linear energy transfer, L_∞ , (often denoted linear energy transfer, LET or L).
1541 Unrestricted linear energy transfer, L_∞ means that the transfer energy includes the energies of
1542 all emitted δ -electrons independent of their range, while restricted LET, L_Δ , means that only
1543 δ -electrons with energies of less than Δ are considered. In general, LET is not simply related
1544 to the biological effectiveness because, for example, ions of different Z show different
1545 effectiveness at the same LET-value (Cucinotta et al., 2000a).

1546 (89) The study of the LET-dependence of RBE is concentrated on the study of radiation
1547 effects in single cells by irradiating thin cell samples with charged particles of various types
1548 and energies. In animal experiments, however, with long ranges of charged particles in the
1549 tissue, the LET-dependence of an effect, e.g. cancer induction, is more difficult to study.

1550 (90) While for neutrons and alpha particles a broad range of experimental data to many
1551 different biological endpoints exists including data from animals (NCRP, 1990; ICRP, 2003),
1552 the situation for high-energy charged particles is more problematic. Epidemiological data on
1553 cancer induction in humans from exposure to high-energy particles and heavy ions are not

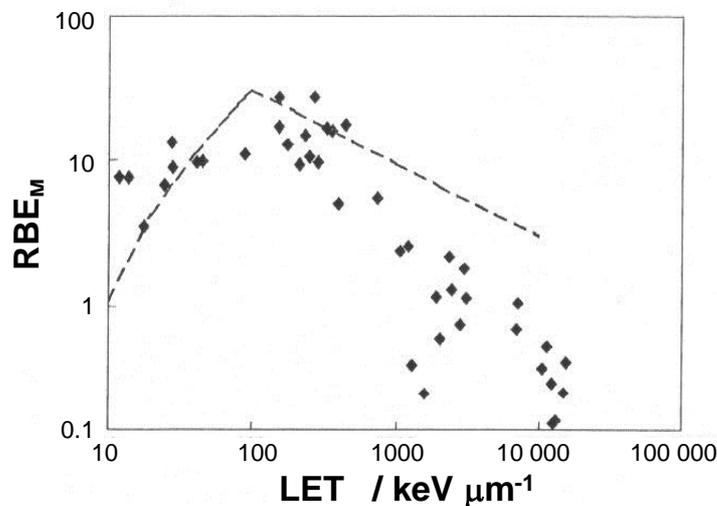
1554 available and experimental data on cancer induction in animals are scarce (ICRP, 2003).
 1555 Most RBE data for high-energy protons and heavy ions have been obtained by experiments
 1556 with cells at high doses ($> 1\text{Gy}$) and high dose rates which are of particular interest for
 1557 heavy-ion radiotherapy applications (Taucher-Scholz et al., 1999, George et al., 2003,
 1558 Durante, 2002) with only a few studies of tumors in mice (Fry et al., 1985, Alpen et al., 1993,
 1559 Weil et al., 2009).

1560 (91) Experimental data of RBE versus LET have been obtained for various biological
 1561 endpoints. The general shape of the RBE-LET relationship is always similar. At low LET-
 1562 values there is an increase in RBE with increasing LET up to about 100 to 150 $\text{keV}/\mu\text{m}$ and
 1563 for higher LET-values, RBE decreases with increasing LET. Figures 3.2 to 3.4 show some
 1564 typical examples of RBE-LET relationships obtained for different biological endpoints.
 1565



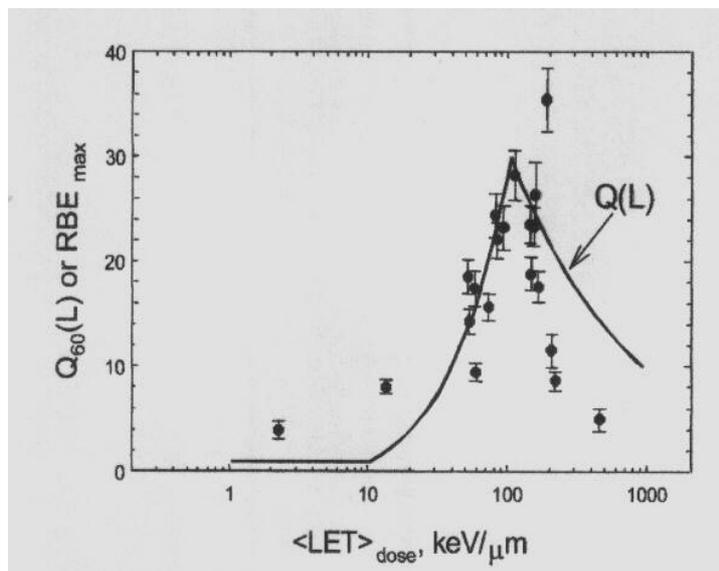
1566 Fig. 3.2. RBE_M versus LET for oncogene transformations in C3H10T1/2 cells of embryos of mice
 1567 (Brenner et al., 1992).
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Fig. 3.3. RBE_M versus LET for the induction of mutations at the hprt locus in Chinese hamster V79 cells. Data are from Kiefer et al. (1999, 2001). The dotted line shows the $Q(L)$ function as defined in ICRP Publication 60 (ICRP, 1991).

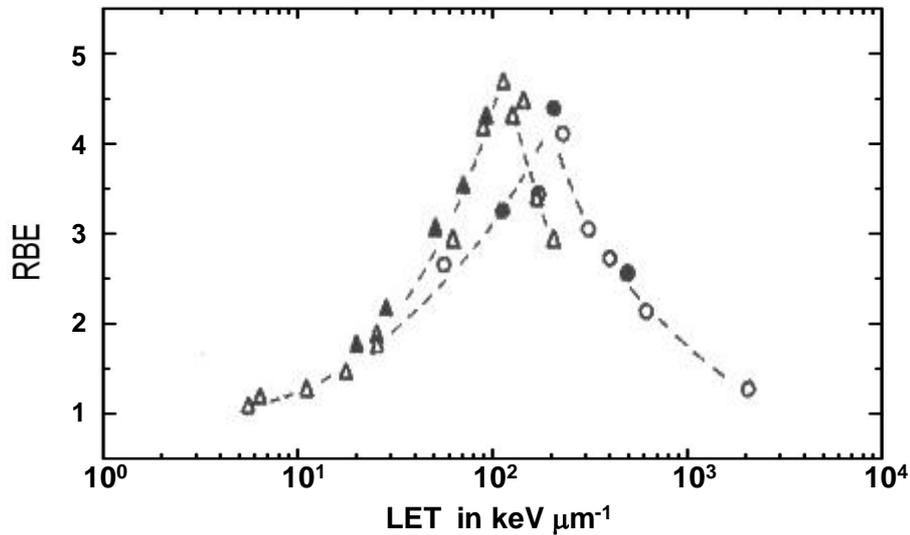


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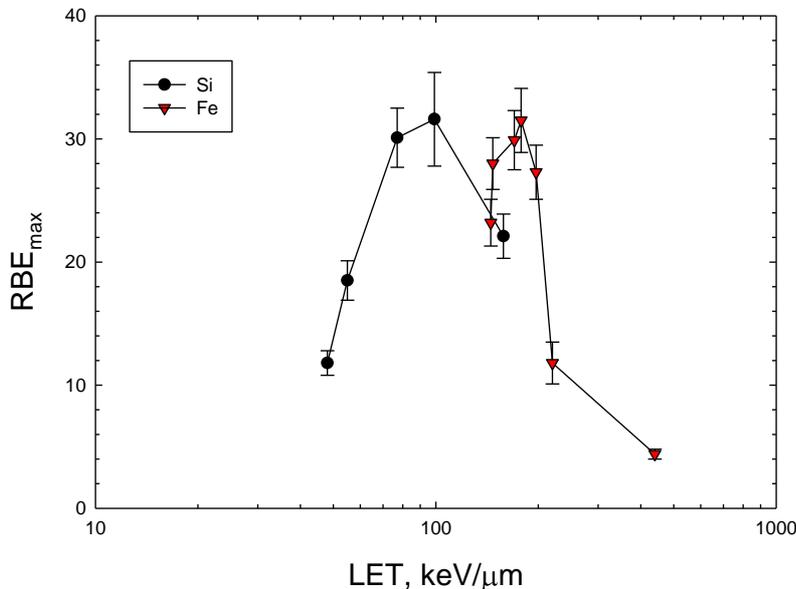
Fig. 3.4. RBE_M versus LET for total chromosomal exchanges measured within the first cell cycle using the premature chromosome condensation (PCC) method (George et al., 2007, Cucinotta et al., 2008). The line shows the $Q(L)$ function as defined in ICRP Publication 60 (ICRP, 1991).

(92) In general, the biological effectiveness is not simply related to LET, especially for heavy ions with high LET-values. This is because the width of a particle track depends on the velocity of the particles. Particles of different Z show different effectiveness at the same LET-value (Cucinotta et al., 2011). The position of the peak RBE versus LET depends on particle charge number and does not occur at a fixed LET (see Figs. 3.5 and 3.6). The peak position changes from less than 100 keV/μm to more than 150 keV/μm as Z increases from protons with $Z=1$ to Fe with $Z=26$. RBE depends on charge Z and energy E of the particle,

1589 and not on LET alone. At a fixed value of LET less than the peak position, particles with
 1590 lower Z are more biologically effective. In addition, the slope of rise of RBE with LET is
 1591 variable with the endpoint/system studied, and the slope of decrease of RBE at high LET-
 1592 values is predicted to be proportional to LET^{-1} rather than $LET^{-1/2}$.
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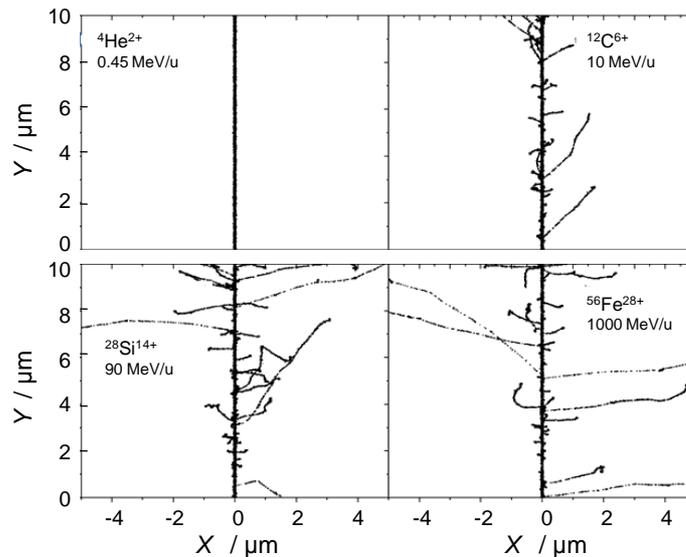
1594
 1595
 1596 Fig 3.5. Relative biological effectiveness (RBE) for inactivation of V79 hamster cells (closed
 1597 symbols) and T1 mammalian cells (open symbols) exposed to beams of accelerated heavy ions (▲, Δ
 1598 deuterons and He-ions; •, ○ heavier ions) versus LET (Thacker et al., 1979).
 1599



1600
 1601 Fig. 3.6. RBE_{max} versus LET for chromosomal aberrations (total exchange) of human lymphocytes
 1602 exposed to Si and Fe nuclei, relative to low-dose and dose-rate γ -rays (George et al., 2007).
 1603

1604 (93) For the same LET-value the distribution of δ -rays strongly varies with the ion
 1605 considered (see Fig 3.7). The approach of describing radiation quality in terms of LET
 1606 assumes that the various ion tracks shown in Fig. 3.7 produce the same cancer risk, although

1607 the initial physical-chemical stages are quite distinct. Hence, there is considerable support to
 1608 relate RBE to a function better correlated to the track structure of the charged particles
 1609 passing tissue (Cucinotta et al., 2011), especially to the spatial distribution of δ -rays along the
 1610 tracks. Arguments from Katz (Katz, 1970; Katz et al., 1972) and observations by Goodhead
 1611 et al. (Goodhead, 1980) supports the hypothesis that especially for heavy ions biological
 1612 effects are strongly influenced by particle track structure including δ -ray effects rather than
 1613 by the stopping power (LET) only.



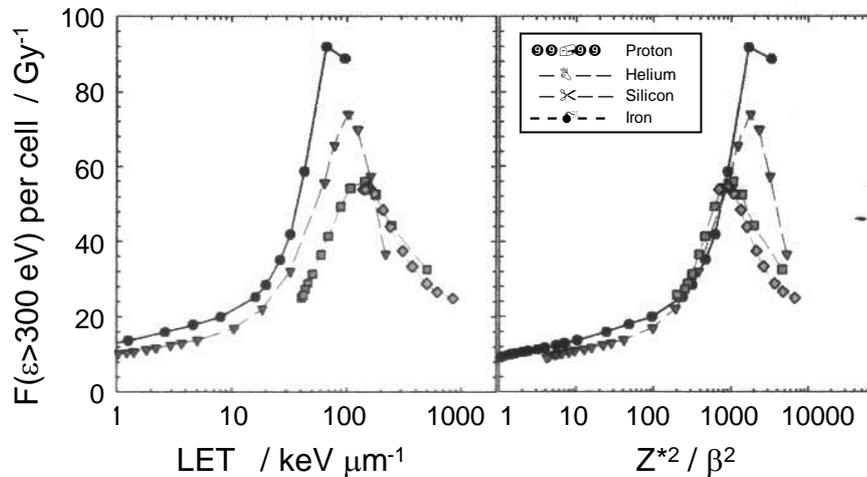
1614 Fig. 3.7. Simulated tracks segments for the following ions in water: ^4He (0.45 MeV/u), ^{12}C (10
 1615 MeV/u), ^{28}Si (90 MeV/u), and ^{56}Fe (1 GeV/u). Shown are projections over the XY plane. Ions are
 1616 generated at the origin along the Y axis in liquid water at 25°C under identical LET conditions (~ 150
 1617 $\text{keV } \mu\text{m}^{-1}$). Each dot represents a radiolytic species (Plante et al., 2008).
 1618
 1619

1620 (94) A parameter Z^{*2}/β^2 , where Z^* is the effective charge number of the nuclei and β the
 1621 velocity of the nuclei relative to the light velocity, has been proposed by Katz and others
 1622 (Katz, 1970; Katz et al., 1972) to be a better descriptor of energy deposition in small volumes
 1623 than LET, especially for heavy ions. This is based on the idea that a reaction cross section
 1624 with respect to biological reactions in tissue should include the effect of δ -rays. A value of Z^*
 1625 can be obtained by using the following equation given by Barkas (Barkas, 1963):

1626
$$Z^* = Z (1 - \exp(-125 \beta Z^{2/3})) \tag{3.7}$$

1627 where Z is the charge number of the nuclei considered. As an example Figure 3.8 shows
 1628 calculations comparing the frequency of energy deposition above 300 eV in a volume of
 1629 about the size of a nucleosome. Obviously the parameter Z^{*2}/β^2 provides an improved
 1630 descriptor of energy deposition in small volumes compared to LET.
 1631

1632



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Fig. 3.8. Number of nucleosomes per cell receiving 300 eV or more as a function of LET (left) or Z^{*2}/β^2 (right). Calculated data are shown for H, He, Si, and Fe nuclei (Cucinotta, 2011).

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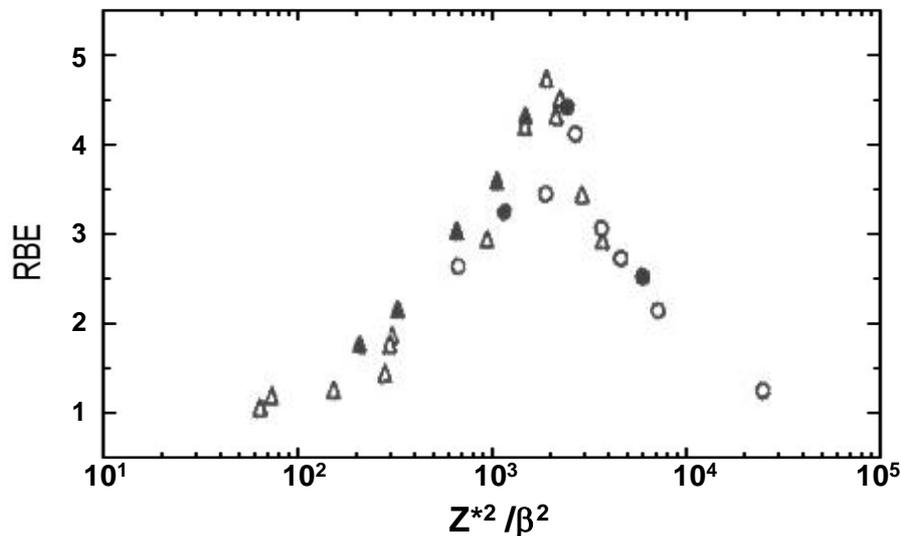
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(95) As a consequence, a RBE function has been derived and described to be dependent on two parameters of the particle, either given by E and Z , or alternatively by Z^{*2}/β^2 (Cucinotta et al., 2011). In Fig. 3.9 the same RBE data as shown in Fig. 3.5 are plotted against Z^{*2}/β^2 and this plot may be a better basis for defining a quality factor function than using the LET parameter.



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Fig 3.9. Relative biological effectiveness (RBE) for inactivation of V79 hamster cells (closed symbols) and T1 mammalian cells (open symbols) exposed to beams of accelerated heavy ions (\blacktriangle , \triangle deuterons and He-ions; \bullet , \circ heavier ions) versus $(Z^*/\beta)^2$ (Thacker et al., 1979).

3.2.2 Radiation weighting factor

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(96) Radiation weighting factors, w_R , have been used in the definition of the protection quantity equivalent dose in an organ or tissue in ICRP Publication 60 (ICRP 1991) and numerically modified in Publication 103 (ICRP, 2007). Values of w_R are given for various

1652 types of radiation either incident on the human body or emitted by radionuclides in the body.
 1653 Data for w_R are given in Table 3.2. The same values of the radiation weighting factors are
 1654 applied to all tissues and organs of the body independent of the degradation of the primary
 1655 radiation and the production of secondary radiations of different radiation quality. It may be
 1656 seen as a mean factor representing radiation quality averaged over the different tissues and
 1657 organs of the body. The application of w_R -values is restricted to low doses and dose rates and
 1658 should not be applied in cases of higher doses where tissue reactions may occur.

1659 (97) Except for neutrons, all types of particles are given a single weighting factor value.
 1660 This simplification is seen to provide sufficient precision for general applications in
 1661 radiological protection, even if it is well known that radiation quality depends also on the
 1662 energy of the particle involved. This is especially the case for heavy ions of high energies
 1663 which, however, in most radiation fields at the Earth are less important of radiological
 1664 protection.

1665
 1666 Table 3.2. Radiation weighting factors¹, w_R (ICRP, 2007)

Radiation type	Radiation weighting factor, w_R
Photons	1
Electrons and muons	1
Protons and charged pions	2
Alpha particles, fission fragments, heavy ions	20
Neutrons	A continuous curve as a function of neutron energy (see equation 3.9 and Fig. 3.2)

1667 (1) All values relate to the radiation incident on the body or, for internal sources, emitted
 1668 from the source.

1669
 1670 (98) For neutrons the following function is recommended for the calculation of
 1671 radiation weighting factors (ICRP, 2007):

$$w_R = \begin{cases} 2.5 + 18.2 e^{-[\ln(E_n)]^2/6} & , E_n < 1 \text{ MeV} \\ 5.0 + 17.0 e^{-[\ln(2E_n)]^2/6} & , 1 \text{ MeV} \leq E_n \leq 50 \text{ MeV} \\ 2.5 + 3.25 e^{-[\ln(0.04E_n)]^2/6} & , E_n > 50 \text{ MeV} \end{cases} \quad (3.8)$$

1672
 1673 where the neutron energy, E_n , is given in MeV (see also Fig. 3.10).
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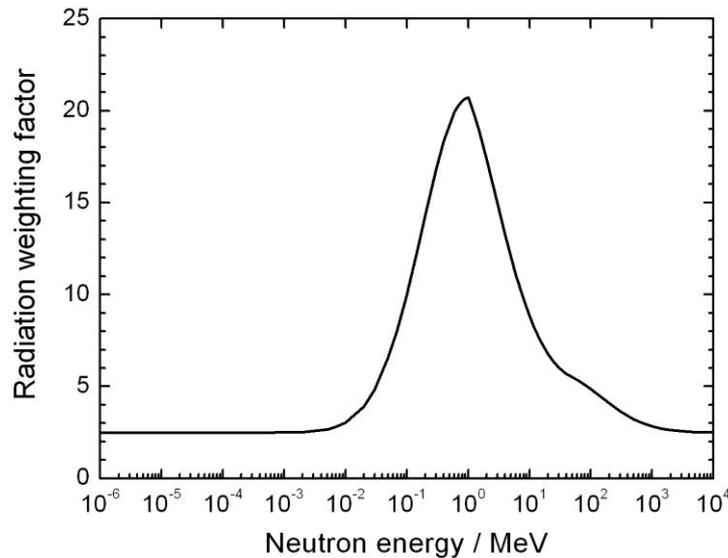


Fig. 3.10. Radiation weighting factor, w_R , for neutrons versus neutron energy (ICRP, 2007).

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(99) For applications in space, where high-energy heavy ions significantly contribute to the total dose in the human body, a more realistic approach for radiation weighting should be chosen (ICRP, 2007). This could, for example, be based on the calculation of mean quality factors in the human body (see Section 3.2.3). Some other difficulties with the w_R -concept for application in high energy radiation fields have also been discussed by Pelliccioni (1998).

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3.2.3 Quality factor

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(100) Radiation quality is seen to be strongly correlated with its energy deposition properties along the tracks of charged particles, especially with the ionisation density along their tracks. For applications in radiological protection, the different biological effectiveness of radiation is considered by introducing the quality factor function, $Q(L)$. $Q(L)$ characterizes the biological effectiveness of a charged particle with a linear energy transfer L at a point of interest in tissue relative to the effectiveness of a reference radiation at this point. Q is defined by a function of L in water (not in tissue) as given in various publications of ICRP and ICRU (ICRP, 1963, 1977, 1991; ICRU, 1970, 1986). RBE-values provide the basis for the selection of a quality factor function used in the definition of the specific dose quantities in radiological protection (see Section 3.2). For this application, all photons and electrons (all low-LET radiation with $L < 10$ keV/ μ m) are weighted by $Q = 1$. This is seen to be an approximation sufficient for usual radiation protection applications and greatly simplifies measurements and calculations.

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1700

(101) The actual quality factor function $Q(L)$ with L for charged particles in water was given in Publication 60 (ICRP, 1991):

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$$Q(L) = \begin{cases} 1 & L < 10 \text{ keV}/\mu\text{m} \\ 0.32 L - 2.2 & 10 \text{ keV}/\mu\text{m} \leq L \leq 100 \text{ keV}/\mu\text{m} \\ 300/\sqrt{L} & L > 100 \text{ keV}/\mu\text{m} \end{cases} \quad (3.9)$$

1706 (102) The $Q(L)$ function is the outcome of radiobiological investigations on cellular and
 1707 molecular systems as well as on the results of animal experiments (see e. g. (ICRP, 2003))
 1708 and has not been changed since 1990. The function includes, however, also some judgments
 1709 for simplifying radiological protection practice. $Q(L) = 1$ for $L < 10$ keV/ μ m, even it is well
 1710 known that the RBE of photons increases with decreasing energy (ICRP, 2003).

1711 (103) The quality factor Q at a point in tissue is then given by:

1712
$$Q = \frac{1}{D} \int_{L=0}^{L=\infty} Q(L) D_L dL \quad (3.10)$$

1713 where D is the absorbed dose in tissue and $D_L = dD/dL$ the distribution of D in L (for
 1714 charged particles in water) at the point of interest in tissue.

1715 (104) For exposure of the human body by neutrons the radiation field is modified in the
 1716 body by moderation of the incident neutrons and secondary radiation from neutron reactions.
 1717 Therefore, for a given neutron exposure situation, the value of the quality factor depends on
 1718 the position in the body and the mean radiation quality factor in organs and tissues of the
 1719 body may differ. For each organ or tissue T, a tissue-mean radiation quality factor, Q_T , can
 1720 be calculated using:

1721

1722
$$Q_T = \frac{1}{m_T D_T} \iint_{m_T L} Q(L) D_L dL dm \quad (3.11)$$

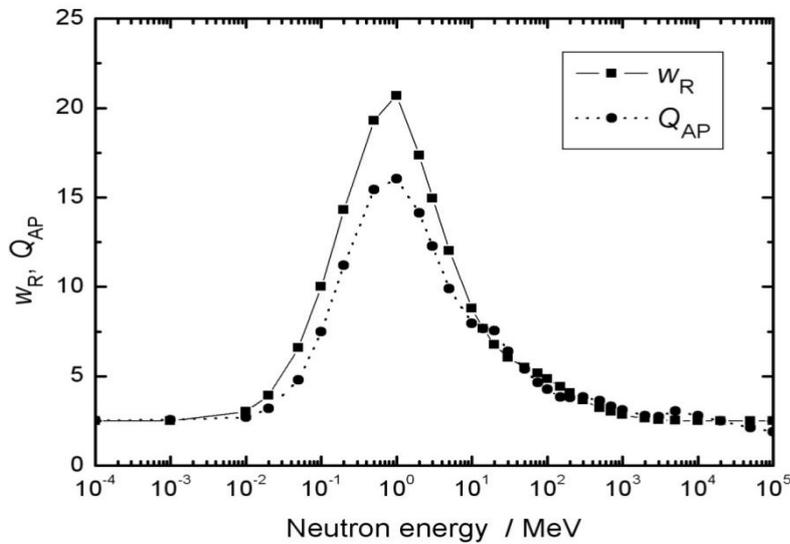
1723 where m_T is the mass of the organ or tissue T. A body-mean quality factor, Q_E , averaged over
 1724 the human body by considering the mean organ absorbed doses, D_T , and the tissue weighting
 1725 factors, w_T , is given by

1726
$$Q_E = \sum_T w_T Q_T D_T / \sum_T w_T D_T . \quad (3.12)$$

1727 (105) Figure 3.11 shows both the radiation weighting factor and the body-mean quality
 1728 factor, $Q_{E,AP}$, calculated for the adult male reference phantom, for monoenergetic neutrons
 1729 (AP incidence) versus neutron energy. Values of Q_E are similar for other directions of
 1730 neutron incidence (e.g. ISO). Obviously stronger differences between w_R and Q_E are only for
 1731 neutron energies between 0.1 MeV and 10 MeV. For neutron energies above 10 MeV, the
 1732 difference is negligible. Differences between Q_T for a single organ and w_R may be, however,
 1733 much larger.

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Fig. 3.11. Radiation weighting factor, w_R , and body-averaged mean quality factor, $Q_{E,AP}$, calculated for neutrons (AP incidence) versus neutron energy (Sato et al., 2009).

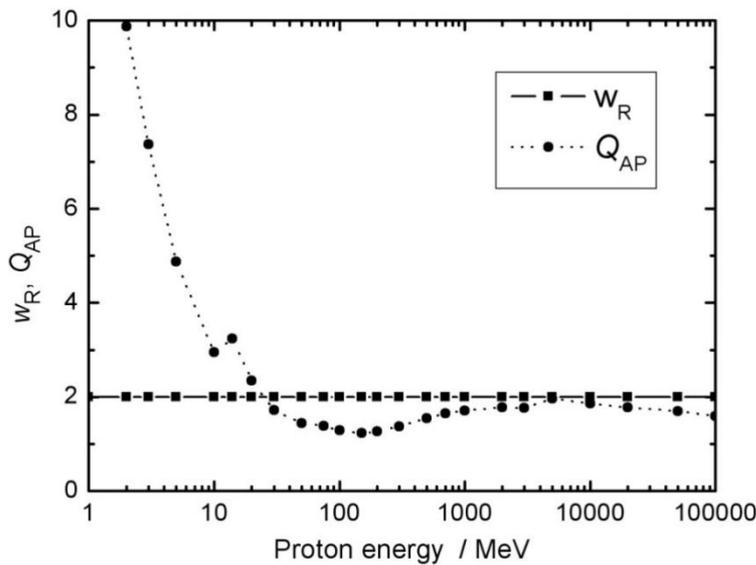
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(106) Similar calculations have been performed for incident protons by Sato et al. (Sato, 2009) and these data are shown in Fig. 3.12.



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Fig. 3.12. Radiation weighting factor, w_R , and body-averaged mean quality factor, $Q_{E,AP}$, calculated for protons (AP incidence) versus proton energy (Sato et al., 2009).

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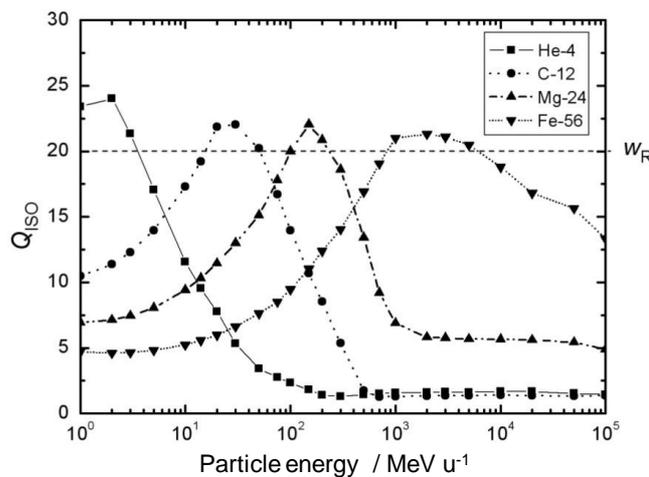
1752

(107) While for energies of incident protons below about 10 MeV there are large differences between the radiation weighting factor and the mean quality factor, they are much lower at higher proton energies and are less than 20% at proton energies above 1 GeV. Protons below about 10 MeV, however, are low-penetrating radiation which are mostly stopped already in the skin of exposed persons and hence contribute little to the effective dose in cosmic radiation fields with many protons of high energies. When exposure of the skin needs special attention, protons below about 10 MeV, however, need also to be

1753 considered. In such cases, the absorbed dose to the skin, and not the effective dose is of prime
 1754 interest.

1755 (108) At proton energies above 20 MeV, the mean quality factor is always between 1 and
 1756 2. At these energies, protons can be seen to be low-LET particles. The increase of Q at
 1757 energies above 150 MeV is due to proton reactions in tissue where secondary charged
 1758 particles are produced.

1759 (109) While for high-energy neutrons and protons the difference between w_R and the mean
 1760 quality factor is relatively small, the situation is quite different for heavy ions.



1761 Fig. 3.13. Radiation weighting factor, w_R , and body-averaged quality factors, $Q_{E,ISO}$, for ^4He , ^{12}C ,
 1762 ^{24}Mg and ^{56}Fe (ISO incidence) versus particle energy (Sato et al., 2010).
 1763

1764
 1765 (110) The body-mean quality factor is strongly varying with the type and energy of the ion
 1766 (Sato et al., 2010), while the w_R -value has been fixed to 20 for all heavy ions and all energies.
 1767 For example, Fig. 3.13 shows body-mean values of Q_E for ^4He , ^{12}C , ^{24}Mg and ^{56}Fe ions with
 1768 isotropic radiation incidence to the body. The value varies between about 2 and 24 depending
 1769 on the ion type and the energy. There are also stronger variations in Q_T depending on the
 1770 position of the organ or tissue in the human body. This situation underlines the decision, not
 1771 to select a single radiation weighting factor value for all heavy ions and all particle energies.
 1772 The recommended concept of quantities for use in radiological protection of astronauts in
 1773 space takes account of this fact (see Section 3.3.1).

1774 (111) Similar to the dependence of RBE for high-energy ions, a different way of defining
 1775 a quality factor function may be derived from the track structure of the charged particles
 1776 rather than being only related to LET. Cucinotta et al. (Cucinotta, 2011a) have proposed a
 1777 functional dependence of a quality factor on two parameters of the particle, Z and E or
 1778 alternatively can be expressed in terms of Z^*2 / β^2 and LET. Because the RBE values for an
 1779 induction of leukaemia and of solid cancers are quite different, they proposed two different
 1780 functions $Q(Z,E)$ for leukaemia and solid cancers (see Fig. 3.14).

1781 (112) The general ideas discussed on track-structure models have been used to define a
 1782 cross section function related to risk which may be written as

1783
$$\Sigma(Z, E) = \Sigma_0 P(Z, E) + (\alpha_\gamma L / 6.24) (1 - P(Z, E)) \quad (3.13)$$

1784 with

1785
$$P(Z, E) = (1 - \exp(-Z^*2 / \kappa\beta^2))^m \quad (3.14)$$

1786 where Σ_0 , m , and κ are parameters which should be based on fits to data from radiobiology
 1787 experiments, and the low-LET slope, α_γ , estimated from epidemiological data for γ -
 1788 radiations. The parameter β is the particle velocity relative to the velocity of light. Z^* is the
 1789 effective charge number which includes a velocity dependent correction to Z at low particle
 1790 energies (Barkas, 1963) as defined in eq. (3.7).

1791 Using the equations above, a quality factor function, $Q(Z,E)$ can then be defined by

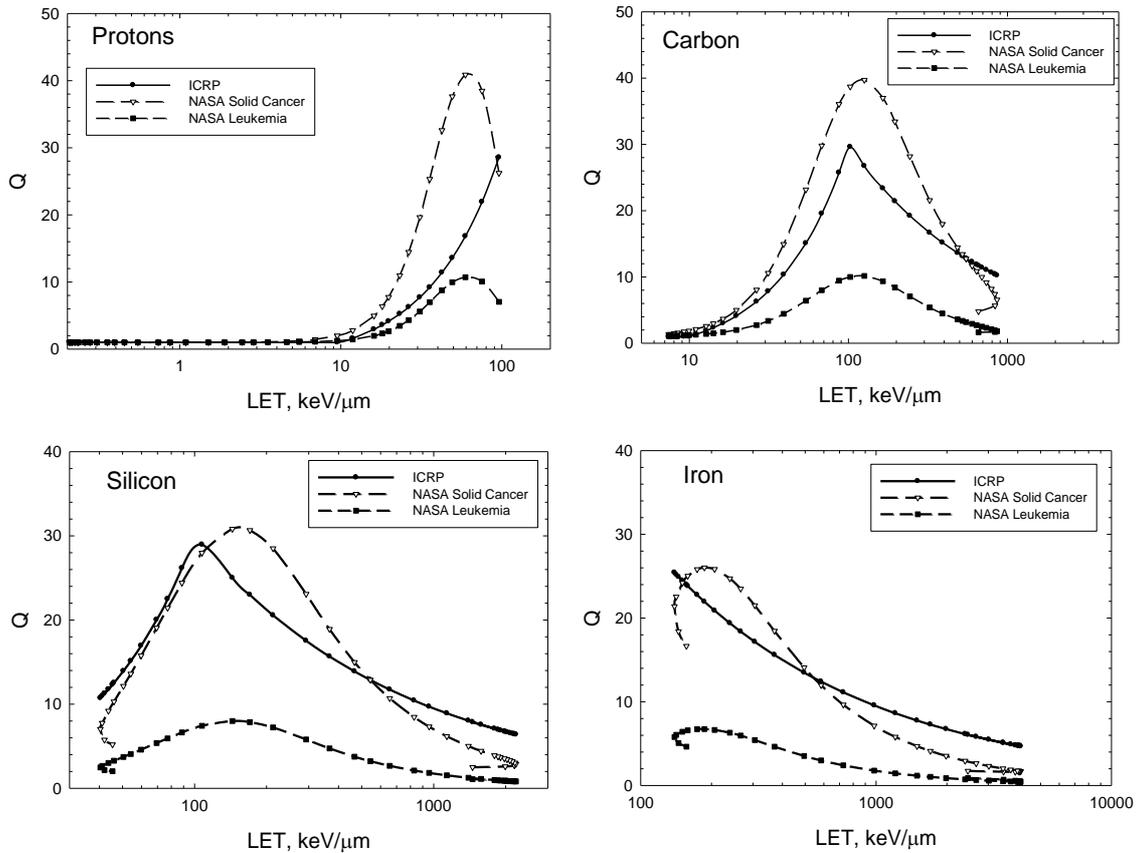
1792
$$Q(Z,E) = (1 - P(Z, E) + (6.25 (\Sigma_0/(\alpha_\gamma L)) P(Z, E) \tag{3.15}$$

1793 (113) Note that L is not an independent variable in addition to Z and E , but for a given Z
 1794 and E , the L -value is fixed. Alternatively, one may write $Q(Z,L)$ or also $Q(E,Z^{*2}/\beta^2)$. At high
 1795 particle energies, this function defines a $1/L$ -dependence of Q different from the $1/L^{1/2}$ -
 1796 dependence given by the $Q(L)$ function of eq. (3.10). The interpretation of the parameters is
 1797 quite general, and is not tied to a particular track structure models per se, but rather is an
 1798 efficient parameterization of radiobiology data for particles. The parameters can be described
 1799 as follows: Σ_0 is the maximum value of the cross section, which is related to RBE_{\max} for the
 1800 most biologically effective particle types, m is the slope of the cross section for increasing
 1801 ionisation density, with values $m>1$ necessary to have $RBE>1$, and κ determines the
 1802 saturation value of the cross section with increasing Z^{*2}/β^2 , where the RBE begins to
 1803 decline.

1804 (114) High-energy protons of about 150 MeV have an LET similar to γ -radiation and their
 1805 kinetic energy is below a value where nuclear reactions become important. Many
 1806 experiments have shown a biological effectiveness for those protons to be very similar to γ -
 1807 radiation. For proton of this energy, it is $P(Z,E) \ll 1$, and $Q \sim 1$.

1808 (115) For solid cancer risks, radiobiology data are sparse. However, the largest RBE for
 1809 HZE nuclei is in the range from 20 to 50 for solid tumors in rodents, and for chromosomal
 1810 aberrations and mutations in human cells. A lower value is found for leukaemia. This
 1811 assumes a linear dose response at low doses for charged particles, ignoring non-targeted-
 1812 effects or other possible mechanisms which may lead to deviations from linearity. Thus, for
 1813 example, if a peak RBE value of approximately 40 is assumed for Si at 100 keV/ μm where
 1814 $P(Z,E) \sim 1$, Σ_0 / α_γ can be estimated as $40 \times 100 / 6.24$. A comparison of the quality factor based
 1815 on LET and on both, L and Z is shown below (Fig. 3.14). The influence in the uncertainties in
 1816 the values of these parameters, which describe the uncertainties in information on RBE_{\max}
 1817 and the peak position of the RBE_{\max} with LET for different particles can be investigated
 1818 using Monte-Carlo methods (Cucinotta et al., 2011).

1819



1820
 1821 □ Fig. 3.14. LET dependence of the quality factor, Q , for H, C, Si, and Fe nuclei as defined by the
 1822 Commission (ICRP, 1991) and as proposed by NASA (Cucinotta, 2011) differently defined for
 1823 considering relative risks of either solid cancer or leukaemia .
 1824

1825

3.3 Approach for space applications

1827

1828 (116) The situation in a space vehicle is characterized by the primary radiation field
 1829 consisting of various high-energy charged particles from protons up to heavy ions such as Fe-
 1830 56 or even higher Z values, and by radiation components of photons, electrons, neutrons and
 1831 other reaction products from the interaction of primary particles with the materials of the
 1832 spacecraft (see Chapter 2). This results in many different types of radiation together with
 1833 broad energy distributions up to particle energies of many GeV/u.

1834 (117) Individual doses are generally higher than in usual exposure situations on Earth. As
 1835 a consequence, deterministic effects in specific organs, e.g. lens of the eye or skin, cannot be
 1836 ignored. This needs specific consideration and will be further discussed in Sect. 3.3.3.

1837 (118) The consequence of this specific situation is that some concepts of the quantities
 1838 used in radiological protection on Earth have to be reconsidered for use in space. For
 1839 example, the operational quantity for area monitoring of penetrating radiation which is based
 1840 on the dose equivalent at 10 mm depth of the ICRU sphere has been mainly designed on the
 1841 basis of photon and neutron data for control of effective dose and is limited in its application
 1842 to radiation with energies where secondary charged particle equilibrium is achieved at about
 1843 10 mm depth in tissue. This is not the case for very high energy particles. For this situation
 1844 computer modelling and simulation of exposure situations become very important in addition

1845 to of measurements.

1846

1847 **3.3.1 Protection quantities**

1848

1849 (119) Radiation risk estimates are generally based on absorbed doses in the tissues and
 1850 organs of the human body and the concept of the mean absorbed dose, D_T , in various organs
 1851 and tissues of the body (see Section 3.1.1) has been assumed to be applicable also for
 1852 astronauts in space. Limitation to this concept, however, may arise by the fact that fluence
 1853 rates of heavy ions are very low. If only few particles pass an organ with a high energy
 1854 transfer along each single particle track, averaging of the dose over an organ is a
 1855 simplification which increases the uncertainty when risk estimates are considered. Another
 1856 problem arises due to the large fraction of charged particles in the radiation field. Depending
 1857 on their energy, they may be stopped in the human body and hence the depth dose
 1858 distribution may not sufficiently homogenous over larger tissues or organs in the body to
 1859 consider only an averaged value. Nevertheless, the mean absorbed dose in organs or tissues
 1860 of the body is a very useful concept for radiological protection practice and for omni-
 1861 directional (isotropic) exposure of an astronaut which can often be assumed in space, this
 1862 problem is less important.

1863 (120) D_T is a quantity which cannot be measured but is usually determined by applying
 1864 calculated conversion coefficients which relate the mean dose in an organ or tissue to an
 1865 external radiation field quantity (fluence for particles or air kerma for photons). They are,
 1866 however, not calculated for any individual, but for adult male and female reference persons
 1867 (ICRP, 2009) and are, therefore, restricted in the assessment of individual doses. The charged
 1868 particles in the energy range of some GeV/u which are present in space, have very long
 1869 ranges in tissue which results in a relatively homogeneous exposure of the human body and
 1870 the variation of the mean organ doses is not very large, especially in case of isotropic
 1871 exposure (see Section 6.3).

1872 (121) Obviously, the general use of $w_R = 20$ for all heavy ions does not reflect the
 1873 variation of RBE with type and energy of heavy ions and the Q -approach is better correlated
 1874 with the assumption of a general dependence of RBE on LET and possibly on (Z^2/β^2) (see
 1875 Section 3.1.3). It is, therefore, endorsed to follow the approach already applied by space
 1876 agencies and to use the term *dose equivalent in an organ or tissue T*, $H_{T,Q}$, defined by

1877
$$H_{T,Q} = Q_T D_T \tag{3.16}$$

1878 with the mean quality factor Q_T in an organ or tissue T for the given radiation field. When
 1879 using the $Q(L)$ function, Q_T is calculated by

1880
$$Q_T = \frac{1}{m_T D_T} \int_{m_T} \int_{L=0}^{L=\infty} Q(L) D_L dL dm \tag{3.17}$$

1881 with the mass, m_T , of the organ or tissue considered.

1882 (122) If a quality factor is defined by a function $Q(Z,E)$, a Q_T -value can be calculated by

1883
$$Q_T = \frac{1}{m_T D_T} \int_{m_T} \left(\sum_Z \int_E Q(Z, E) D_E(Z, E) dE dm \right) \tag{3.18a}$$

1884 or alternatively

1885
$$Q_T = \frac{1}{m_T D_T} \int (\sum_Z \int_L Q(Z, L) D_L(Z, L) dL dm) \quad . \quad (3.18b)$$

1886 (123) Similar to equivalent dose in an organ or tissue, H_T , the value of dose equivalent in
 1887 an organ or tissue is defined for organs and tissues in males and females by:

1888
$$H_{T,Q}^M = Q_T^M D_T^M \quad \text{and} \quad H_{T,Q}^F = Q_T^F D_T^F \quad . \quad (3.19)$$

1889 (124) In most cases the difference of Q_T for males and females is small and a tissue-mean
 1890 quality factor, Q_T , may be used for both sexes.

1891 (125) Based on the definition of effective dose, E , the effective dose equivalent, H_E , can
 1892 then be calculated by applying the tissue weighting factors, w_T , as given in ICRP Publication
 1893 103 (ICRP, 2007)

1894
$$H_E = \sum_T w_T H_{T,Q} \quad (3.20)$$

1895 where for $H_{T,Q}$ the mean value from doses for the male and female phantom is chosen. Note
 1896 that this quantity has already been defined by the Commission in Publication 26 (ICRP,
 1897 1977) but with different tissue weighting factors and replaced by effective dose in Publication
 1898 60 (ICRP, 1991).

1899 (126) An application of effective dose equivalent, however, is not recommended for the
 1900 assessment of doses of individual or small groups of astronauts when these should become a
 1901 basis for risk estimates. The recommended w_T values are single values for both sexes and also
 1902 based on data for persons of all ages including children. They are not appropriate for a
 1903 realistic risk assessment for male and female astronauts and hence risk estimates should be
 1904 based on either absorbed dose or dose equivalent data for the organs and tissues of males or
 1905 females respectively, and corresponding risk factors for these tissues for male and female
 1906 adults (see e.g. Table A.4.19 in Annex B of ICRP Publication 103 (ICRP, 2007)). If
 1907 necessary, also the age of the person considered may be taking into account. Even the quality
 1908 factor may differ depending if leukaemia or solid cancer induction or mortality is considered
 1909 (see Fig. 3.14).

1910 (127) If, nevertheless, a value of effective dose equivalent is needed for recording, H_E
 1911 should be calculated by:

1912
 1913
$$H_E^M = \sum_T w_T H_{T,Q}^M \quad \text{or} \quad H_E^F = \sum_T w_T H_{T,Q}^F \quad (3.21)$$

1914 depending on the sex of the astronaut.

1915
 1916 **3.3.2 Operational quantities**

1917
 1918 (128) Radiation monitoring in a spacecraft and individual monitoring for each astronaut is
 1919 a necessary measure for radiological protection in space and the assessment of mission doses
 1920 of astronauts.

1921 (129) On Earth, area monitoring in terms of the quantity ambient dose equivalent, $H^*(10)$,
 1922 provides in fields of penetrating radiation the information which exposure in terms of
 1923 effective dose a person would receive when staying for a given time at the position of the
 1924 area monitor. In most external fields of strongly-penetrating radiation on Earth, only low-
 1925 LET radiation, mostly x- and γ -radiation, partially also electrons, is present and in few cases
 1926 only neutrons are also important for radiological protection. As a consequence, most area
 1927 monitors used in radiological protection measure either photon or neutron doses and the total

1928 ambient dose equivalent is then achieved by adding both dose components. Concentrating on
1929 photons, electrons and neutrons only has strongly influenced the idea of defining a special
1930 measurable dose quantity in an appropriate simple phantom (the ICRU sphere) for area
1931 monitoring and the assessment of effective dose.

1932 (130) The consequence of the specific situation in space is that some concepts of the
1933 quantities used in radiological protection on Earth have to be revisited. For example, the
1934 operational quantity for area monitoring of penetrating radiation which is based on the dose
1935 equivalent at 10 mm depth of the ICRU sphere in aligned fields has been mainly designed on
1936 the basis of photon and neutron data for control of effective dose and is limited in its
1937 application to radiation with energies where secondary charged particle equilibrium can be
1938 achieved at about 10 mm depth in tissue. This is not the case for very high energy particles.

1939 (131) In radiation fields in space with its large spectrum of different types of particles of
1940 very high energies the definition of $H^*(10)$ is inappropriate (ICRP, 2012) and it will be
1941 difficult to define a dose quantity independent from the human body with the same properties
1942 as ambient dose equivalent for photon and neutron radiation of conventional energy. In
1943 addition, monitoring and assessment of doses in the human body during missions of
1944 astronauts may not be restricted to effective dose equivalent and dose equivalent to the skin,
1945 the lens of the eye and the extremities, but should include other organs and tissues of the
1946 body, too. Hence no specific dose quantity for area monitoring in space has been defined up
1947 to now. The monitors used serve mainly as instruments for recording the environmental
1948 radiation outside or inside a spacecraft and for warning in cases of very intensive SPE's.
1949 They measure particle fluence, LET-distributions or absorbed doses in detector materials (see
1950 Chapter 4). These data are used as input or validation data for calculations of doses in the
1951 human body.

1952 (132) In individual monitoring, the situation is similar to area monitoring and a dosimeter
1953 calibrated in terms of $H_p(10)$ in photon or neutron radiation fields on Earth and worn on the
1954 body of an astronaut in space will not automatically provide an estimate of effective dose
1955 equivalent in complex space radiation fields.

1956 (133) A specific operational dose quantity for individual monitoring in space has not been
1957 defined by ICRU or ICRP. Different quantities and procedures may be applied for an
1958 assessment of organ doses or effective dose equivalent (see Chapter 4). A combination of
1959 measurements of absorbed dose and LET-distributions at the surface of the body may become
1960 an appropriate way for individual dose assessment (see Sect. 4.3.3). Data from area
1961 monitoring within the spacecraft combined with calculated dose conversion coefficients can
1962 also be used to calculate organ dose equivalents or effective dose equivalents for persons
1963 present in that radiation field (see Chapter 6). This, however, needs the knowledge of the
1964 fluence and energy distribution of all components of the radiation field which in addition may
1965 vary with time. Furthermore application of methods of biological dosimetry may provide a
1966 comparative way of assessing mission doses of astronauts (see Sect. 4.3.4 and 6.4.2).

1967 (134) Specific attention is needed for the measurement of doses from low-penetrating
1968 radiation, e.g. electrons or solar low-energy protons, which may contribute significantly to
1969 doses to the skin and the lens of the eye especially in situations when astronauts are working
1970 outside the spacecraft or during large solar flare events.

1971

1972 3.3.3 Quantities for high doses

1973

1974 (135) At high doses near to thresholds where deterministic effects (tissue reactions) may
1975 occur, organ dose equivalent and effective dose equivalent should generally not be used for

1976 an assessment of radiation risks. While $H_{T,Q}$ and H_E may give an indication, if such dose
1977 range is reached, risk estimates should be based on mean absorbed doses in organs or tissues,
1978 maximum dose values in organs or tissues or on organ dose equivalent depending on the
1979 actual situation and the availability of risk factors for the different types of radiation and
1980 doses involved. Generally tissue reactions occur above a threshold dose in the range of 0.5 - 2
1981 Gy.

1982 (136) The mean absorbed dose in an organ or tissue, D_T , and the RBE weighted mean
1983 absorbed dose, $RBE \cdot D_T$, when high-LET radiation is involved, is the appropriate quantity for
1984 assessing risks of deterministic effects at higher doses. The RBE-value to be chosen may
1985 depend on the organ or tissue considered and the specific dose and dose rate as well on the
1986 type and severity of the tissue reaction considered. In some cases of deterministic effects,
1987 however, not only the mean dose in an organ or tissue but also a local dose in that tissue may
1988 become important (e.g. local skin dose).

1989 (137) The dose limits for tissue reaction are given in terms of the mean absorbed dose in
1990 an organ or tissue, D_T , and this value weighted by an appropriate RBE should be applied. The
1991 RBE values may be taken from ICRP Publication 58 (1989), where values of 6 (range 4 to 8)
1992 for neutrons of energy 1 MeV to 5 MeV, of 3.5 (2 to 5) for neutrons of energy 5 MeV to 50
1993 MeV, of 2.5 (1 to 4) for heavy ions, and of 1.5 for protons are recommended.

1994 (138) Measurements of particle fluence, LET-distributions or absorbed doses can be
1995 performed in any radiation field, even in strong radiation fields producing high doses, e. g.
1996 during solar particle events, where non-stochastic tissue reactions cannot be completely
1997 excluded. Measurements of $H_p(d)$ which is defined by using the fixed $Q(L)$ relationship may
1998 also be performed in such intense radiation fields for the purpose of dose recording. For risk
1999 assessment, however, it must be considered that the $Q(L)$ function has been defined based on
2000 RBE_{max} data at low doses and, therefore, its application in radiation is usually limited to the
2001 low-dose range.
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4. METHODS OF MEASUREMENT OF RADIATION FLUENCES AND DOSES

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(139) Radiation transport calculations, measurements, and interpretation of data are essential components for operational radiation safety. The measurements include an assessment of the environment at the exterior surface of the spacecraft or habitat; assessment of the interior environments in the spacecraft/habitat and EVA suit; the use of personal dosimeters; and the determination of the transmission of radiation to internal organs or tissues.

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(140) Individual monitoring of occupationally exposed persons on Earth is mainly performed to ensure that the exposure is below legal dose limits which are generally set to limit exposure risks to an acceptable level. While the basic limits are given in terms of effective dose, it is generally accepted that at doses far below the limits the measured personal dose equivalent appropriately assesses effective dose in routine radiological protection applications (ICRP, 2007). The situation in space, however, is significantly different. Doses to astronauts in space may be much higher than annual dose limits for occupationally exposed persons on Earth, especially in long duration missions. It is, therefore, important to obtain more precise information about doses and exposure risks. For planning purposes and for operational radiation safety programs, risk assessment is seen to be as important as dose recording for astronauts in space. For any risk assessment, however, the knowledge of the radiation incident on the human body and/or the assessment of doses in the human body are basic preconditions.

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(141) Calculations are essential to mission planning in that they provide pre-flight estimates of the doses that would be received by the astronauts during the different phases of a proposed mission. They are also an important part of the dose assessment process for determining the doses received by the astronauts during the mission.

2031

4.1 Measurement quantities

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(142) The objective of the adopted dose assessment approach in radiological protection is generally to determine the specific protection quantities defined for the low-dose range for limiting the probability of stochastic effects and for avoiding deterministic effects (tissue reactions) in the human body. For applications on Earth these quantities are the equivalent dose in an organ or tissue, H_T , and the effective dose, E (see Section 3.1). For the special situation of astronauts in space, the quantities mean absorbed dose in an organ or tissue, D_T , and dose equivalent in an organ or tissue, $H_{T,Q}$, are proposed for use instead (see Section 3.3). At higher doses when deterministic effects may occur, a differently weighted absorbed dose is required. In this case, a mean value of the relative biological effectiveness, RBE, needs to be determined from the information on the radiation field components and the specific RBE values of the different types of particles involved. There is an important role for biodosimetry. Individual biomarker measurements on astronauts can give consolidation for other dosimetric approaches.

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(143) The radiation environment external to a spacecraft in low Earth orbit consists of electrons, positrons, neutrons, protons and all stable atomic nuclei (up to charge $Z = 92$). Particle energies range from a few eV for trapped electrons and from thermal neutrons to 10^{14} MeV for GCR (see Chapter 2). Most of the electrons will not penetrate the wall of the spacecraft, but could penetrate the space suits worn during extravehicular activity (EVA),

2051 resulting in doses to the skin and lens of the eye. Nuclear interactions of neutrons, protons
2052 and heavier nuclei with spacecraft, space suits, the Earth's atmosphere and the human body
2053 produce secondary radiation, which add to the radiation field. The techniques of radiation
2054 monitoring vary according to particle type, energy, and the measurement location (inside or
2055 outside the spacecraft, inside EVA suits). The radiation environment can be classified on this
2056 basis:

- 2057 • Trapped electrons – outside spacecraft and inside EVA suit;
- 2058 • Trapped protons (< 10 MeV) – do not penetrate spacecraft or EVA suit;
- 2059 • Protons and light charged particles (> 10 MeV) – outside and inside spacecraft, and
2060 inside EVA suit;
- 2061 • GCR and secondary photons – outside and inside spacecraft, and inside EVA suit;
- 2062 • Secondary charged-particles – inside spacecraft and inside EVA suit;
- 2063 • Neutrons - outside and inside spacecraft, and inside EVA suit.

2064
2065 (144) In addition to variations of the primary external radiation field the relative
2066 contribution of each component (including the secondary radiation) to organ and tissue
2067 absorbed doses and dose equivalents at each location will also vary according to additional
2068 factors, including the mass distribution inside the spacecraft and the EVA suit construction.

2069 (145) Mean absorbed doses and dose equivalent in organs and tissues of a human body are
2070 generally not directly measurable. An approach to the estimation of these quantities is given
2071 in Chapter 6. These include (1) calculations of particle type and energy and direction
2072 distributions of fluence in radiation fields at the location of an astronaut plus the application
2073 of organ absorbed dose and dose equivalent conversion coefficients; (2) direct assessment of
2074 organ absorbed doses and dose equivalents for an astronaut by radiation transport calculation
2075 using energy and direction distributions of fluence from outside of the spacecraft, or
2076 otherwise, at the astronaut's location; and (3) measurement of absorbed dose or dose
2077 equivalent near or on the astronaut and the use of results from calculations applying
2078 anthropomorphic phantoms.

2079 (146) The main objectives of environmental measurements are the provision of radiation
2080 field data of particle types, fluences and microdosimetric quantities, absorbed doses and dose
2081 equivalents, using detectors in assemblies of various sizes, both integral and differential (with
2082 respect to time, or LET, or energy, or direction, as appropriate) and, in some cases,
2083 normalized to calculations of the radiation field components. For personal monitoring, the
2084 same quantities might be determined, but more importantly data are required for the
2085 determination of absorbed dose and dose equivalent. Absorbed dose and dose equivalent
2086 values measured by personal dosimeters worn on the body can be applied as estimates of D
2087 and H at a point in adjacent tissue, or can be used with prior data as estimates of absorbed
2088 dose or dose equivalent to specific organs and tissues at larger depth. Environmental
2089 monitoring can provide radiation field data as input for calculations which will yield
2090 estimates of doses in the human body or may even be used to directly assess individual risks.
2091 It may measure dose quantities such as absorbed dose, D , the distribution of D in L , $D(L)$, or
2092 dose equivalent, H . It can also provide measurements in support of in-flight dose
2093 management, dose recording, and ALARA actions.

2094 (147) The radiation detectors determine a particular measurement quantity. These include
2095 (1) particle type and energy and direction distributions of fluence; (2) dose deposition in the
2096 detector material; and (3) other dosimetric quantities such as LET or lineal energy, y . In
2097 addition there are calibration data on detector response and normalization of detector
2098 response (for example normalized to equivalent water absorbed dose for a radiation type and

2099 energy)
2100 (148) Several types of instrumentation are available including both active and passive
2101 devices. Active instrumentation should have a time resolution sufficient to identify temporal
2102 variations in the radiation field. Alarm or warning capabilities of instruments can be used to
2103 support in-flight implementation of dose management and ALARA actions. Passive detectors
2104 provide an integration of the exposure information over long time intervals. They are usually
2105 very robust, small in their dimensions and need no power supply. Some devices also allow
2106 on-demand readout. For all instrumentation used for measurements in space the following
2107 must be established: (1) full response characterization and calibration; (2) measurement
2108 model; and (3) knowledge of uncertainties. Comparison of instrument responses is also
2109 useful. Many details of instrumentation for dosimetry in space have been published in review
2110 papers by Benton (2001), Badhwar (2002), and Caffrey and Hamby (2011). Further
2111 information has been presented by the HAMLET collaboration (HAMLET, 2011).
2112

2113 **4.2 Purpose of measurements**

2114
2115 (149) Measurements are performed for the purpose of determining individual exposures,
2116 monitoring changes of the radiation environment and performing ALARA. Instrumentation
2117 positioned inside or outside of a spacecraft or habitat can provide essential data for the
2118 characterization of the primary fields impinging on the spacecraft, such as galactic radiation,
2119 the trapped particle radiation field and the field caused by solar particle events. Such data
2120 from outside allow corrections to be made to the input data of transport codes to determine
2121 the field inside the spacecraft or habitat and so can reduce dose assessment uncertainties. Ion
2122 chambers or other active detectors with well-defined wall thickness can serve to monitor the
2123 short-term variations in the electron environment for EVA.

2124 (150) Selectively located active and passive instrumentation allows radiation monitoring
2125 and further adjustments to the calculated internal environment. Particle spectrometers can
2126 provide data for the evaluation of transmission factors and of response functions of
2127 dosimetric devices and allow evaluation of calculated values and uncertainties of dosimetric
2128 quantities. Both active and passive devices are useful for determining absorbed dose,
2129 absorbed dose distribution over LET, absorbed dose distribution over lineal energy, and
2130 estimates of dose equivalent. Personal dosimeters worn by astronauts can determine some of
2131 these quantities at adjacent tissues.

2132 (151) There are short-term and long-term variations depending on solar activity, which
2133 modify the environmental radiation field (components, particle fluences, energies and
2134 direction distributions), so that energy- and direction-sensitive instrumentation is mandatory.
2135 In addition, active instrumentation allows the time-resolution of trapped, solar, and GCR
2136 components.
2137

2138 **4.3 Instrumentation for radiation spectrometry, area and personal monitoring**

2139 **4.3.1 General**

2140
2141
2142 (152) No single device can determine the required dose quantities for all components of
2143 the radiation field. Since there are large variations in the relative contributions to total
2144 absorbed dose and dose equivalent from the different particle types, it is not generally
2145 possible to determine the absorbed dose or dose equivalent from just one component (or few
2146 components) and apply a correction factor to determine the absorbed dose or dose equivalent

2147 for the entire radiation field.

2148 (153) The dosimetry may be simplified by considering separately the radiation in terms of
2149 its energy deposition properties characterized by LET (or lineal energy). These include both
2150 low-LET charged particles (those of LET less than 10 keV/μm) and high-LET charged
2151 particles (those of LET equal to or greater than 10 keV/μm). High-LET particles may be
2152 separated into the high-energy heavy ions ($Z > 2$) and their fragments, and the mainly high-
2153 LET particles produced by the strong-force interactions of neutrons and high-energy protons.
2154 Often separate dose measurements are performed for these categories, preferably minimizing
2155 any overlap of response to avoid “double counting”. The choice of measurement devices is
2156 dictated by radiation response characteristics (dependence on particle type and energy and on
2157 the quantity to be determined), operational characteristics (direct determination of dose
2158 quantities, input to model calculations, desired accuracy, dose management and ALARA
2159 support), as well as practical issues such as reliability, robustness and availability. The
2160 addition of devices measuring y and L distributions of energy deposition in tissue from all
2161 particles has substantially improved the situation of dosimetry in space.

2162 (154) Another important consideration is the time required for analysis of the measured
2163 data compared with the duration of the mission. It might be desirable for long duration
2164 missions that there are read-out facilities for passive devices on board.

2165 (155) Additional passive personal dosimeters may be needed on the astronauts during
2166 EVA missions to take account of the spatial variations in the degree of shielding provided by
2167 the spacesuit and the spacecraft.

2168 (156) The development of a set of coefficients that directly relate dose equivalent obtained
2169 with TLDs and NTDs at the surface of a body to mean organ absorbed dose or dose
2170 equivalent for the space radiation environment is desirable in principle. It is, however, a very
2171 difficult task due to the complexity of the radiation field in space. While it might be possible
2172 for the GCR component, it seems to be impossible for trapped radiation owing to its strong
2173 variation with time and location.

2174 (157) Neutrons are indirectly ionising particles. They are generated by interactions of
2175 GCR and higher-energy protons with the atmosphere of Earth, or other body, the spacecraft
2176 or habitat, or within fellow astronauts. The energy distribution of those neutrons has several
2177 maxima due to reactions producing neutrons from the incident GCR nuclei or the target
2178 atoms in spacecraft materials or tissue. The lowest energy maximum occurs as about 1 MeV
2179 from neutrons produced from the nuclear evaporation process from target atoms in
2180 interaction with GCR. A second broader maximum occurs at about 100 MeV from knockout
2181 and cascade reactions leading to fast neutrons from emitted from target nuclei. At higher
2182 energies, there is an even broader maximum near about 1000 MeV due to neutrons produced
2183 from the GCR nuclei by either evaporation or knockout and cascade reactions. Various
2184 methods are applied for neutron monitoring in space (see e.g. Benton et al., 2001). To detect
2185 absorbed dose to tissue and dose equivalent from the high-LET secondary particles from
2186 neutrons, and for similar particles produced by strong-force interactions of higher energy
2187 protons, it is necessary to use detector materials and local shielding that closely matches
2188 tissue elemental composition.

2189

2190 **4.3.2 Active devices**

2191

2192 (158) Active devices record and display data in real time, or near-real time. This enables
2193 measurements of fluence or dose rates as well as time integrated values. Most active
2194 instruments can be turned on or off so as to operate in specific circumstances such as EVA

2195 and during an SPE. Active detectors require electrical power that can be provided through
2196 connections with power supplies in the spacecraft or through batteries. During recent years a
2197 broad range of different types of active instruments has been used for radiation measurements
2198 on board of the ISS either for scientific investigations (see Table 4.1) or for operational area
2199 monitoring (see Table 4.2). In the following some types of instruments are described.
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2201
2202
2203

Table 4.1. Active radiation detectors applied on board the ISS for scientific applications during recent years, some of them in combination with the MATROSHKA phantom (Dettmann et al., 2007).

Instrument	Ref.	Heritage	Measured parameters
DOSMAP	(Reitz et al., 2005)	International collaboration	Various parameters
DOSTEL – Silicon Telescope	(Reitz et al., 2009)	Christian Albrechts University Kiel, German Aerospace Center, Cologne	LET distrib., absorbed dose, dose equivalent
SSD – Silicon Scintillator Device	(Reitz et al., 2009)	Christian Albrechts University Kiel, German Aerospace Center, Cologne	Absorbed dose, neutron dose, organ dose
LIULIN – Silicon Detectors	(Dachev et al., 2006)	Solar Terrestrial Influences Laboratory, Bulgaria	Absorbed dose, dose rate
ALTCRISS – Silicon strip detector	(Casolini et al., 2007)	INFN and University of Rome Tor Vergata, Rome, Italy	Particle energy distrib. up to Iron nuclei, LET distribution, dose equivalent
ALTEA – Silicon strip detector	(Fuglesang, 2007; Narici et al., 2004)		
BBND – Bonner Ball Neutron Detector	(Koshiishi et al., 2007)	Japan Aerospace Exploration Agency, JAXA, Japan	Neutron energy distrib. and neutron dose

2204
2205

Table 4.2. Active and semi-active radiation detectors used on board the ISS for area monitoring.

Instrument	Ref.	Heritage	Measured parameters	
Tissue equivalent proportional counter (TEPC)	(Badhwar et al., 1994)	NASA Johnson Space Center, Houston	LET distribution, absorbed dose, dose equivalent	
Charged particle detector system (IV-CPDS)	(Lee et al., 2007)		NASA Johnson Space Center, Houston	LET distribution, particle energy distribution, Nuclear abundances up to Oxygen
Charged particle detector system (EV-CPDS)				
Ionisation Chamber (R-16)	(Benghin et al., 2008)	Moscow State University, Moscow	Absorbed dose, dose rate	
Silicon detector units (DB8)		Space Research Institute, Bulgaria	Absorbed dose, dose rate	
TL – system (PILLE)	(Apáthy et al., 2002; 2007)	KFKI, Hungary	Absorbed dose, dose rate	

2206
2207

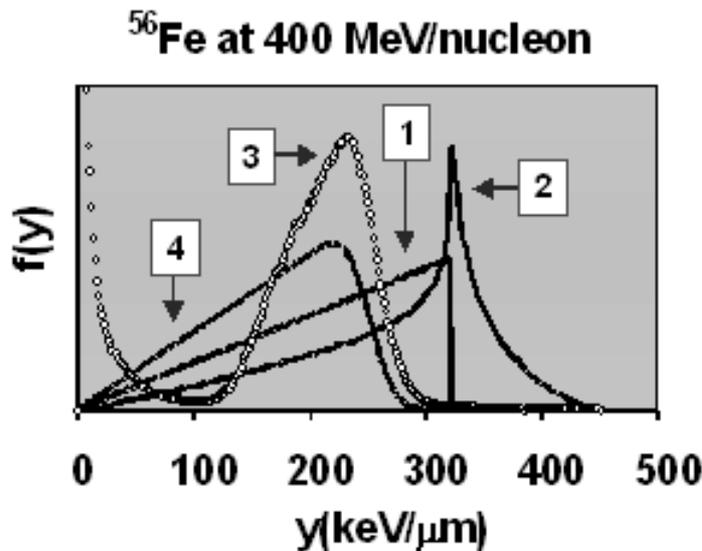
Tissue equivalent proportional counters

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(159) A tissue equivalent proportional counter (TEPC) is a low-pressure proportional counter with a wall of tissue-equivalent material especially designed for use in microdosimetry and radiological protection. The pressure of the gas in the detector is chosen to simulate a volume of tissue comparable to the dimensions of the nuclei of a mammalian cell. The charge produced by an ionising particle traversing the chamber volume is collected

2214 and, by calibration of the detector, the electrical signal can be related to the energy loss of the
 2215 crossing particle in that simulated tissue volume. Data are recorded on an event-by-event
 2216 basis such that one can obtain a distribution of energy deposition events in a small element of
 2217 tissue with a known covering material in terms of lineal energy, y , which can be correlated
 2218 with LET, and therefore characterizes the ionisation density along tracks of particles and
 2219 radiation quality. The lineal energy, y , is defined by the quotient ε/\bar{l} , where ε is the energy
 2220 transferred to the chamber gas by ionisation by a crossing particle and \bar{l} the mean path
 2221 length of the charged particles in the chamber volume. Consideration should be given to
 2222 factors which influence performance, such as wall effects.

2223 (160) The response to photons and neutrons depends on the probability of the production
 2224 of secondary charged particles in the wall passing the detector volume - and hence on the
 2225 wall material. Mostly materials that are tissue-equivalent with respect to neutrons are chosen,
 2226 for example A-150 plastic. For incident charged particles, each particle crossing the chamber
 2227 volume produces a signal. Depending on the shape of the volume, even for incident
 2228 monoenergetic charged particles a broad y -distribution is obtained. The y -distribution is
 2229 further modified by straggling effects and secondary charged particles produced in the
 2230 chamber wall (see Fig. 4.1).
 2231



2232 Fig. 4.1. Frequency distribution, $f(y)$, of lineal energy in a TEPC for a uniform fluence of ^{56}Fe ions.
 2233 Curves 1 and 2 are calculated by taking the chord length in a sphere (curve 1) and a cylinder (curve 2)
 2234 times the LET of the Fe-ion. Data of Curve 3 are from measurements with a spherical detector and
 2235 Curve 4 is the results of a calculation using restricted LET, L_{Δ} , and considering straggling (NCRP,
 2236 2002).
 2237

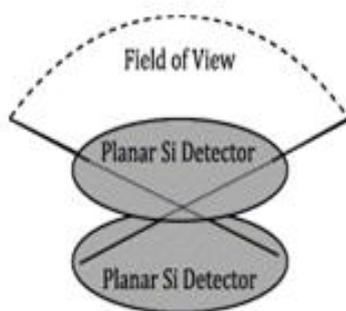
2238 (161) The term $y \cdot f(y)$ as a function of y approximates the dose distribution in terms of LET
 2239 and can be used to determine a mean quality factor for the absorbed dose to the detector
 2240 configuration (Doke et al., 2001). When the data are integrated over the distribution of y or
 2241 L , the TEPC can generate absorbed dose and absorbed dose rate. Data from a TEPC can be
 2242 displayed continuously or stored for later transmission to mission control. Special TEPC
 2243 systems have been designed for use in spacecraft (see e.g. Caffrey et al., 2011).
 2244

2245 **Semiconductor devices**
 2246

2247 (162) Semiconductor devices for the detection of charged particles are thin diodes, mostly
 2248 Si-diodes (thickness of about 50 to 5000 μm), that record the energy deposited by a charged
 2249 particle in the sensitive volume of the detector. For particles with normal incidence and fully
 2250 traversing the sensitive detector volume, the ratio of the deposited energy to the thickness of
 2251 the detector yields the approximate LET for the incident particle in that material. If charged
 2252 particles from various directions pass the detector, a mean path length in the sensitive
 2253 detector volume need to be determined for assessing an LET-distribution. Thus, a single
 2254 detector can approximately provide an estimate of the charged particle fluence distribution in
 2255 linear energy transfer and time. This distribution can be integrated to yield dose and dose rate
 2256 for protons and heavier charged particles in the detector material, and by the use of
 2257 conversion coefficients, determine dose to tissue. Often some detectors are used as a
 2258 telescope which restricts the response to a smaller solid angle.

2259 (163) Several multi-array solid-state detectors are combined to form a particle telescope
 2260 that measures both energy deposition and flight direction. When these data are combined, the
 2261 detector yields a more accurate estimate of LET in the detector material, and thus the
 2262 distribution of fluence in LET, direction, and time. Using appropriate conversion coefficients,
 2263 the data can also be used to obtain $D(L)$ distributions and Q -values for heavy charged
 2264 particles, but with the restriction that the incident particles originate from a fixed direction
 2265 depending on the orientation of the detector. Because of size limitations, this type of detector
 2266 is only sensitive over a restricted solid angle.

2267 (164) The energy lost in solid-state detectors of a telescope can also be used to identify the
 2268 particle charge as well as the incident energy to ultimately obtain the distribution of fluence
 2269 in energy, particle charge number and time. Several of these detectors can be combined to
 2270 point in different directions to provide a more complete description of the radiation field
 2271 either outside or inside a spacecraft.
 2272



(a)

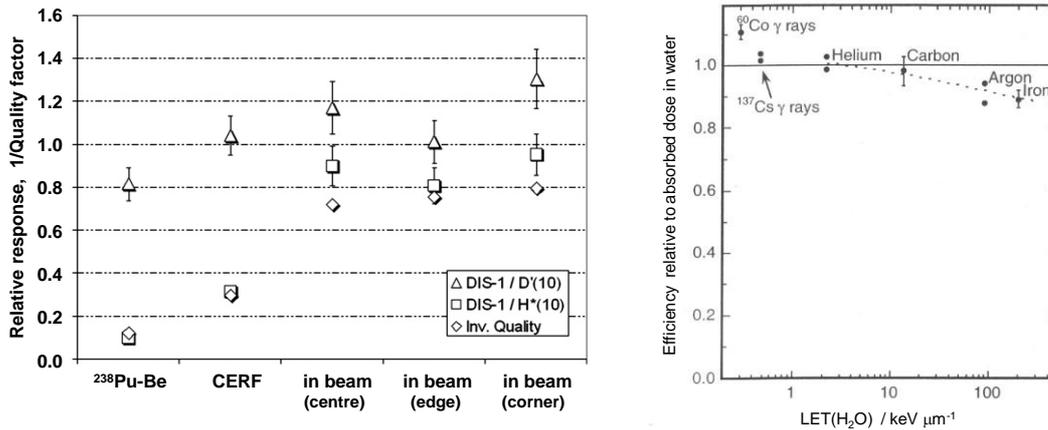
(b)

2273
 2274 Fig. 4.2. Charged particle spectrometer DOSTEL designed for space applications (Posner et al.,
 2275 2276 2005). Schematic view (a) and realized detector (b) (Beaujean et al., 2002).
 2277
 2278

2279 Direct ion storage devices

2280 (165) This device combines an ionisation chamber with a semiconductor device.
 2281 Specifically, the direct ion storage dosimeter, DIS, is based on coupling a gas-filled ion
 2282 chamber with a semiconductor non-volatile memory cell where the charges produced in the
 2283 chamber volume are stored. These are compact integrating devices that can be read out
 2284 periodically without deleting the information and are used to estimate accumulated doses

2285 over periods of several hours to at least one year. The response of the DIS to charged
 2286 particles incident on the detector and passing the chamber volume is approximately equal to
 2287 the energy deposited, while the response to photons and neutrons incident on the DIS and
 2288 producing secondary charged particles in the detector depends on the material and thickness
 2289 of the chamber wall. The response to low-energy charged particles incident on the DIS
 2290 depends also on the chamber wall thickness due to their energy loss in the wall.
 2291



2292
 2293 Fig. 4.3. Relative response of the DIS-1 dosimeter.

2294 (a) Dosimeter reading in terms of personal dose equivalent, $H_p(\text{DIS})$, normalised to absorbed dose in
 2295 10 mm depth of tissue, $D(10)$, (triangles), and ambient dose equivalent, $H^*(10)$ (squares). The inverse
 2296 of the quality factor, $1/Q$, is also indicated (diamonds) (Otto, 2010).

2297 (b) Efficiencies of DIS-1 relative to absorbed dose in water as a function of $\text{LET}(\text{H}_2\text{O})$. Each point
 2298 for ⁶⁰Co-γ radiation, carbon and iron ions represents an average value from six samples (1 s.d.). Two
 2299 of the samples were irradiated with each of three doses (10 mGy, 25 mGy, and 50 mGy in water).
 2300 Points for ¹³⁷Cs-γ radiation, helium ions and argon ions represent two samples irradiated with a dose
 2301 of 10 mGy. (Yasuda, 2001).
 2302

2303 **Bonner sphere spectrometer**

2304 (166) In principle, a Bonner sphere spectrometer consisting of a set of Bonner spheres of
 2305 different size is well suited for measuring neutron fluence and spectral distributions of
 2306 neutrons. On ground, neutrons over a wide energy range are often monitored by such
 2307 moderator-based survey instruments whose responses have also been extended to higher
 2308 energies by introducing a layer of heavy metal in their moderator (see e.g. Wiegel et al.,
 2309 2000). However, those instruments are not suitable for use in spacecraft because of their
 2310 heavy weight as well as their high sensitivity to HZE particles. A Bonner Ball Neutron
 2311 Detector (BBND) was used on Space Shuttle flight (Matsumoto et al., 2001) and on ISS
 2312 (Koshiishi et al., 2007) for measuring the fluence of neutrons with energies below about 15
 2313 MeV. However, also high-energy neutrons are present in spacecraft (see par. 158). Thus,
 2314 development of a new instrument having a light weight and less sensitivity to HZE particles
 2315 is necessary for precisely monitoring neutron fluence or doses in spacecraft.
 2316

2317 **Electron detectors**

2318 (167) Active electron detectors need to be specially configured to measure low-LET
 2319 radiations, in particular, electrons below about 1 MeV. Electrons of these energies are
 2320 normally not an issue inside the spacecraft but could be of concern during EVA, since
 2321 electrons above a few hundred keV can penetrate the spacesuits. Since the trapped electron

2322 fluence rate can change by many orders of magnitude during and following a large magnetic
2323 storm, due to short-term perturbations of the geomagnetic field, it is recommended that an
2324 active detector sensitive to electrons be installed outside the spacecraft to serve as a monitor
2325 for fluctuations in the electron component of the space radiation environment to enable dose
2326 management during EVAs. Such a monitor could be a simple ionisation chamber or solid-
2327 state detector with a wall thickness sufficient to strongly attenuate very low energy electrons
2328 but thin enough to record electrons that could penetrate a spacesuit (Evans et al., 2008).

2329

2330 **Active personal dosimeters**

2331 (168) Active personal dosimeters (APD) can give an instant indication of both
2332 accumulated dose and dose rate. Preset visual and audible alarms are also provided, so that
2333 these devices can be used simultaneously as an integrating dosimeter and as an alarm
2334 dosimeter. APD can be used as supplementary dosimeters to a passive dosimeter used for
2335 routine dosimetry. It is clear that for low-LET radiation the energy and directional response
2336 characteristics of APD are, in most cases, as good as passive dosimeters, able to measure
2337 doses in continuous radiation fields with acceptable accuracy.

2338 (169) An ideal personal dosimeter would be active, store integrated dose data and dose
2339 rate time profiles, and respond to all field components allowing a sufficiently precise
2340 determination of absorbed dose rate and dose equivalent rate to adjacent tissues. Most
2341 available electronic personal dosimeters have been designed to measure absorbed dose to
2342 tissue under a defined covering layer from photon and beta radiation and their high-LET
2343 response characteristics are not well determined. Such dosimeters might be considered for
2344 the measurement of the low-LET component of the fields in spacecraft/habitat. However,
2345 even if used only to determine the low-LET component, a full characterization of their
2346 charged-particle and neutron response is necessary. Pixel-based detectors can be used as
2347 personal dosimeters. They can provide energy deposit distributions, energy and charge for all
2348 particles. They are based on a read out chip that embeds the electronics for each pixel within
2349 the pixel's footprint. Careful calibration is necessary as for other devices.

2350 (170) If active personal dosimeters are not used, then it may be necessary to develop on-
2351 board readout capabilities for passive dosimeters, especially on long-duration missions in
2352 space. Although on-board readout of nuclear etched track detectors is not feasible, on-board
2353 systems for readout of TL- and OSL-dosimeters already exist.

2354

2355 **4.3.3 Passive devices**

2356

2357 (171) Passive devices can be used for both area monitoring and personal monitoring. No
2358 single passive device is capable of dose measurement across the full energy and direction
2359 distribution of particles available in space. The instrumentation and detectors should be
2360 designed to have optimum performance in the types of particles noted above, namely: low-
2361 LET charged particles (*i.e.* $L < 10 \text{ keV } \mu\text{m}^{-1}$), including such particle produced by non-
2362 ionising particles incident on the instrumentation; high-LET charged particles from neutrons
2363 and protons that undergo strong-force interactions; and high-LET heavy charged particles
2364 (HZE). While thermoluminescence detectors (TLD) radiophotoluminescence glasses (RPL),
2365 and optically stimulated luminescence dosimeters (OSLD) are used mostly for photon and
2366 electron dosimetry, as well as for neutron dosimetry, plastic nuclear etched track detectors
2367 (PNTD) are well suited for measurements in neutron and heavy charged particle fields. In
2368 space applications, measurement systems combining both detector types are often used in
2369 personal dosimetry.

2370 **Luminescence detectors**

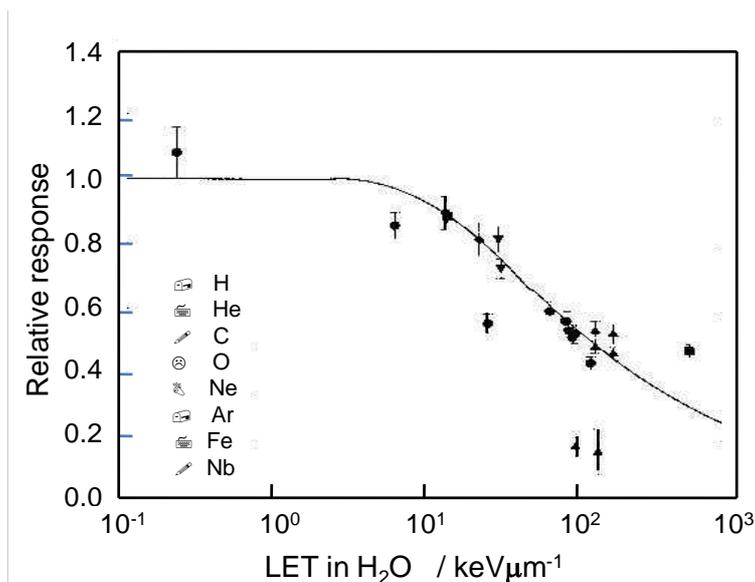
2371 (172) Passive dosimeters currently in use are mostly thermoluminescence detectors
 2372 (TLD). TLDs are small sintered chips or pellets of crystals which show luminescence after
 2373 exposure to ionising radiation. By fast controlled heating of the crystal, the stored energy is
 2374 released as light emitted. The function between the actual temperature and the intensity of the
 2375 emitted light (the glow curve) shows various peaks, the heights of which are proportional to
 2376 absorbed dose. Various crystal materials doped with different elements are in use for personal
 2377 dosimetry.

2378 (173) An alternative to TLD is the use of radiophotoluminescence glasses (RPL). Small
 2379 glass elements respond as do TLDs to incident ionising radiation, storing energy in
 2380 metastable electronic levels. The energy is released as light when the glass is exposed to ultra
 2381 violet laser illumination.

2382 (174) Optically stimulated luminescence dosimeters (OSLD) have also be proposed. In
 2383 OSLD, the stored energy is released by optical laser stimulation. TLDs, RPL glasses, OSLD
 2384 material, or similar types of dosimeters, will have to be fully characterized for the specific
 2385 space environment.

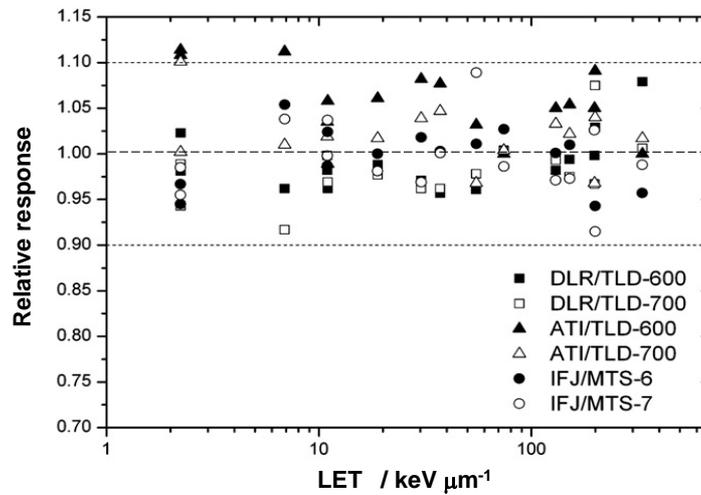
2386 (175) There might be a need for some passive elements to be read out frequently on board,
 2387 and others that are read only on return to Earth. The mode of readout of the luminescence
 2388 dosimeters should be carefully considered.

2389 (176) TLDs, RPL glasses, and OSLDs measure absorbed dose in their material and are
 2390 mostly calibrated in terms of absorbed dose in tissue or water in ¹³⁷Cs or ⁶⁰Co gamma
 2391 radiation reference fields. The detector response, relative to the response to the reference
 2392 radiation depends on the ionisation density around the track of a traversing particle. While for
 2393 $L < 10$ keV/μm there is mostly only a small L -dependence of the response, TLDs, RPL
 2394 glasses, and OSLDs show a strong LET-dependence above about 10 keV/μm, generally with
 2395 decreasing response with increasing LET (see Fig. 4.4). This needs to be well known for
 2396 dosimetry in space radiation fields. Fig 4.5 shows the relative variation of the response of
 2397 some TLDs to cosmic radiation and ions with different LET-values in the detectors.
 2398



2399 Fig. 4.4. Relative response of TLDs for various charged particles. The relative response (relative to
 2400 ⁶⁰Co-γ rays) of peak 5 from TLD-600 and TLD-700 versus mean LET (in water) in the detector is
 2401 shown (Benton et al., 2000; Berger et al., 2006).
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Fig. 4.5. Relative response of various TLDs (normalized to the mean value) to cosmic radiation and ion beams with different LET values in the detector; current results of the intercomparison of personal dosimeters (HAMLET) (Bilski et al, 2011).

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Nuclear track detectors

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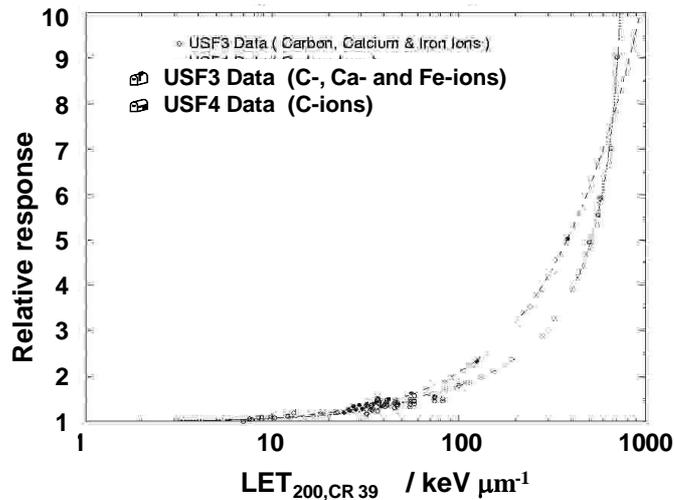
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(177) To detect the high-LET component of the radiation field, plastic nuclear etched track detectors (PNTD) and nuclear emulsions can be employed. Damage to the material is caused by the passage of a charged particle, and these particle tracks can be viewed microscopically, either before or after being realized by suitable chemical treatment. Etched track detectors are generally insensitive for radiation with an LET in water below about $10 \text{ keV } \mu\text{m}^{-1}$ (depending on material) and may be employed in a method that provides D , $D(L)$, or D averaged over certain ranges of L , and H . With suitable detector sets, etched track detectors and nuclear emulsions can determine the charged particle type and its direction in the detection medium, and can allow an analysis of the radiation field at the position of the detector. The analysis of the response of polyallyldiglycolcarbonate (PADC), commonly named CR-39, in terms of the dependence on LET allows the determination of absorbed dose and dose equivalent to a small element of tissue from all charged particles above about $10 \text{ keV } \mu\text{m}^{-1}$ (Zhou et.al., 2006). The detectors can separate the heavy charged particle and neutron contributions by looking at the particle ranges. A further separation procedure to detect the neutron dose component can be made using thin detectors and coincidence techniques or by using of a thin PADC detector and a combination of electrochemical and chemical etching. Stacks of PADC detectors can be used for spectrometry of HZE particles (see e.g. Gunther et al., 2002)



2427 Fig. 4.6. Relative response of PADC detectors (CR-39) as a function of restricted LET, L_{200}
 2428 (O’Sullivan et al., 1999; Zhou et al., 2008).
 2429
 2430

2431 **Superheated emulsions**

2432 (178) Superheated emulsion detectors, sometimes referred to as superheated drop or
 2433 bubble detectors, are small droplets of a liquid above its normal boiling point suspended in a
 2434 viscoelastic medium. The droplets remain in the liquid phase until a charged particle interacts
 2435 inside or near the surface of the droplet. This transfers energy to the droplet and may cause
 2436 local evaporation. If sufficient energy has been transferred and a critical radius is exceeded,
 2437 all the liquid in the droplet will be vaporized and the bubble becomes visible. Superheated
 2438 emulsion detectors respond to neutrons and to heavy-charged particles. A clear calibration
 2439 procedure is required. Recent data on bubble detector responses are presented by Lewis et al.
 2440 (Lewis et al, 2012).

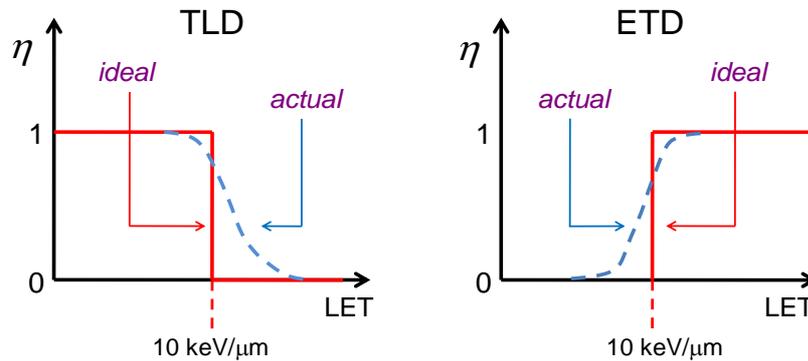
2441 **Combined detector systems**

2442 (179) A combination of one detector for the measurement of the low-LET component and
 2443 one for the high-LET component can allow the determination of dose equivalent in the
 2444 complex radiation field in space. For passive dosimetry, with a package of TLD (or OSLD)
 2445 and PNTD, the dose equivalent in adjacent tissue near the surface of the body is determined
 2446 by:
 2447

2448
$$H = D_{TLD} - \int_{L>10\text{keV}/\mu\text{m}} \eta_{TLD}(L)D_{TLD}(L)dL + \int_{L>10\text{keV}/\mu\text{m}} D_{ETD}(L)Q(L)dL - \int_{L<10\text{keV}/\mu\text{m}} \eta_{ETD}(L)D_{ETD}(L)dL \quad (4.1)$$

 2449

2450 where D_{TLD} is the total absorbed dose recorded by the TLD (equivalent also for OSLD).
 2451 $D_{PNTD}(L)$ is the absorbed dose distribution in LET determined by the etched track detector in
 2452 the high-LET range ($L > 10 \text{ keV } \mu\text{m}^{-1}$) and for which Q is dependent upon L . The correction
 2453 terms consider the non-zero response of the TLD. $\eta_{TLD}(L)$ is the relative dose response in the
 2454 region $L > 10 \text{ keV}/\mu\text{m}$ and $\eta_{ETD}(L)$ that of the PNTD, respectively, in the region
 2455 $L < 10 \text{ keV}/\mu\text{m}$, so that intermediate LET components are not considered twice. It is
 2456 recognized that this requires the verification of the LET-dependence of both the low-LET and
 2457 high-LET detector system, and the elimination of parts from overlapping of responses.
 2458



2459
2460

2461 Fig. 4.7. Schematic relative dose responses of a TLD and a PNTD.

2462

2463 (180) An alternative method is to obtain data on the y or L distribution of energy
2464 deposition from a TEPC or a particle spectrometer used as a passive area dosimeter, and to
2465 apply this for correcting a measurement with a low-LET detector to obtain the full dose
2466 equivalent at the dosimeter's location. For such a correction the efficiency function, $\epsilon(L)$, of
2467 the low- y or low-LET detector needs to be known, and an effective efficiency to be
2468 determined.

2469

2470 4.4 Biomarkers of exposure

2471

2472 (181) Biomarkers of health effects can be divided into 3 categories (Brooks, 1999):
2473 biodosimetry which is a measure of exposure, biomarkers of risk, and biomarkers of the
2474 presence of disease. The term biodosimetry in this context refers to a biological measurement
2475 to assess a biologically based dose equivalent. Because of the complex nature of the space
2476 environment, including nuclear and atomic processes occurring within tissues, and the large
2477 uncertainties in conversion of doses to risk, biodosimetry is routinely performed on the small
2478 population of astronauts involved in International Space Station (ISS) missions, and would
2479 likely be used in future space missions. There are also limitations in the size of devices that
2480 can be worn and practical obstacles to wearing a physical dosimeter at all times in space,
2481 which make biodosimetry an attractive supplement to physical dosimetry. To be useful as a
2482 biodosimetry method, the radiation quality dependence of the response of the assays should
2483 be similar to the $Q(L)$ relation from Eq. (3.10). Biodosimetry assays would likely be carried
2484 out on lymphocytes from a routine blood sample (see e.g. IAEA, 2011). Biomarker assays
2485 may require samples from other tissues including skin, swabs from the oral cavity to collect
2486 cells, urine or bone marrow. There are several biodosimetric methods that can be used for
2487 space travel which are summarized next.

2488

2489 Micronuclei

2490 (182) Micronuclei result from loss of whole chromosomes or acentric chromosome
2491 fragments from daughter nuclei following cell division. They appear as small, membrane-
2492 bounded inclusions in the cytoplasm. Their assessment is relatively easy and rapid and by
2493 including cytochalasin B in the cell cultures analysis of cells is restricted to those in their 2nd
2494 post-exposure cycle. This has greatly increased the sensitivity of the assay. If individual pre-
2495 exposure levels of micronucleus frequency are available, absorbed doses as low as 50 mGy
2496 can be detected; without knowledge of the background frequency the detection level is

2497 estimated to be 100 mGy. A modification of the assay uses a fluorescent pan-centromere
2498 probe to distinguish two types of micronuclei - those derived from either whole chromosomes
2499 or from chromosome fragments. By restricting analysis to the fragment type much of the
2500 background noise in the assay can be reduced (Vral et al., 1997).

2501 (183) The frequency of micronuclei has been used for estimating exposures for a number
2502 of radiation accidents with reasonable agreement with physical dose estimates. The
2503 drawbacks are that few studies have been performed with high-LET exposures, and the assay
2504 is not reliable for partial body exposures. Micronuclei are unstable structures meaning that
2505 their frequency reduces with time due to the turnover of lymphocytes. This makes the assay
2506 unsuitable as a biodosimeter in astronauts exposed to space radiations over prolonged
2507 missions.

2508

2509 **Acentric fragments in prematurely condensed chromosomes (PCC)**

2510 (184) Interphase chromosomes can be prematurely condensed to be observable
2511 microscopically either by fusion with mitotic cells or by treatment with a phosphatase
2512 inhibitor (calyculin A) (Blakely et al., 2003). Structural and numerical chromosomal
2513 alterations can be observed as an increase (or decrease) in centric chromosome number or
2514 from the presence of acentric fragments. The method has been used in a laboratory setting to
2515 assess exposure to x-rays and gamma radiation. The advantages are that non-dividing cells
2516 can be used for assay and the analysis is quite straightforward. The current drawback is that
2517 the assay has had no application as a true biodosimeter, and issues such as responsiveness to
2518 partial body exposures or chronic exposures have not been addressed. Further investigation
2519 appears warranted for assessing the value of PCC as a biodosimeter in the space environment.

2520 (185) The fact that this PCC method has not been used in biodosimetry means that a lower
2521 detection level has not been established. A reasonable assumption is that it will be no more
2522 sensitive than other cytogenetic methods, given similar cell numbers analyzed. The
2523 advantage is a greater ease in obtaining large numbers of cells for analysis. As with
2524 micronuclei, this signal also is unstable with time due to the replacement of lymphocytes
2525 from the stem cell pool.

2526

2527 **Chromosomal aberrations**

2528 (186) The method that has been most extensively used in biodosimetry is that of the
2529 analysis of structural chromosome aberrations. The incorporation of fluorescence in situ
2530 hybridization (FISH) techniques (Cornforth, 2001; Edwards, 2005) has enabled the
2531 assessment of symmetrical (transmissible) translocations. Unlike the assays described above,
2532 these are stable aberrations that pass successfully through cell division and are more suited to
2533 prolonged space missions. This allows for reliable dose estimation at long times after
2534 exposure. Chromosome aberration dosimetry has been successfully applied to radiation
2535 accident victims, A-bomb survivors and a wide range of occupationally and medically
2536 exposed persons including astronauts (George, et al., 2001; Cucinotta et al., 2008). These
2537 scenarios include chronic exposures and partial body exposures for which specific methods
2538 have been developed to aid in exposure estimation. The distribution of aberrations among
2539 cells is useful in this regard. Even with this strong record of success, the application of the
2540 method for dosimetry in astronauts in space has some drawbacks as evidenced by published
2541 studies (reviewed in Testard and Sabatier, 1999; and by Obe et al., 1999). One problem is the
2542 inter-individual variation in response which, however, can be overcome by individual
2543 calibration and determination of background in advance. The response is generally low due to
2544 the nature of the exposures by which only a small number of cells will actually be traversed.

2545 This is especially the case for heavy charged particles. On the other hand, the effective area
2546 (or volume) of the blood system which is distributed over the whole body, is much larger
2547 than that of a passive dosimeter – biodosimetry sees the rare events better than a small
2548 physical dosimeter.

2549 (187) Additional laboratory investigation is required in order to consider using cytogenetic
2550 analysis as a reliable dosimeter for space missions. Biodosimetry assays take skilled
2551 technicians to implement. The level of skill and time, however, to analyze data is no more
2552 demanding than physical dosimetry and the costs of biodosimetry are lower than physical
2553 dosimetry if the launch mass and crew time involved for physical dosimetry is considered.

2554 (188) The detection of low doses using chromosomal aberrations is dependent upon the
2555 number of cells analyzed and whether an individual calibration curve is made prior to the
2556 exposure of interest. This point is addressed by Bauchinger (1995) who estimates that for
2557 5,000 cells analyzed (using a generalized background frequency for dicentrics) a significant
2558 increase in dicentrics should be observed at about 100 mGy for a group of individuals.
2559 Twenty thousand cells would need to be analyzed to detect 50 mGy. Knowledge of the pre-
2560 exposure dicentric frequency can lower this detection level without increasing the number of
2561 cells analyzed.

2562 (189) The study by Tucker et al. (1997) reports a lower detection level of about 500 mSv
2563 of effective dose for occupationally exposed individuals using FISH analysis of stable
2564 aberrations. The exposure was chronic in this case; a situation for which this type of
2565 translocation analysis is particularly applicable. Again, the sensitivity of the assay would be
2566 increased by having available the pre-exposure aberrations frequency. It might be further
2567 increased if the dose response to a reference radiation is determined using a blood sample
2568 obtained prior to space missions (George, et al. 2001). The number of painted chromosomes
2569 used also impacts the sensitivity of the assay. The older studies painted only one or two
2570 chromosomes. Newer studies involved a larger number of chromosome paints or painting the
2571 entire genome which increases the sensitivity. Tucker has recently reviewed the low-dose
2572 sensitivity of the FISH translocations assay (Tucker, 2008).

2573

2574 **Other biomarker methods**

2575 (190) Several other assays as biomarker approaches have been proposed or recently
2576 developed. These include complementary DNA (cDNA) arrays to measure gene expression
2577 (Mezentsev et al., 2011), prolonged life span of erythrocytes bearing transferrin receptors
2578 on their membrane (Gong et al., 1999), gene mutation assays, and electron paramagnetic
2579 resonance (EPR) spectroscopy. A review on EPR dosimetry with tooth enamel is given by
2580 Fattibene and Callens (Fattibene et al., 2010). For these methods to be applied to space
2581 biodosimetry, the dose, radiation quality and duration of the signal over many months will
2582 need to be understood. A more recent assay is the measurement of telomere length.
2583 Telomeres are the short repetitive DNA regions that cap the end of chromosomes protecting
2584 them from deterioration. Reduction in the average length of the telomere has been associated
2585 with several late effects including cancer, neurological disorders, and aging (Schoeftner et al.,
2586 2009). Measurement techniques include southern blot, Q-FISH (Williams et al., 2011), and
2587 flow cytometry. These latter two assays allow rapid analysis of many cells. However, the
2588 radiation quality and dose response of the assay to protons and heavy ions irradiation will
2589 have to be studied further before it can be considered for a deployable biodosimeter for space
2590 missions.

2591

2592 **4.5 Instrument characterization and calibration**

2593

2594 (191) For instrument characterization and calibration in a laboratory, it is necessary to
2595 clearly specify the relevant calibration conditions, including the characteristics of the
2596 reference radiation source, the irradiation facility, and the conversion coefficients used.
2597 Periodic, accurate calibrations are essential, as are meaningful instrument response
2598 comparisons. Calibration (ISO, 2007; JCGM, 2008a) covers a number of the procedures
2599 included in a type test. In the first step, a series of calibration factors (or calibration
2600 coefficients, see below) or responses may be determined for a set of reference conditions,
2601 usually a set of radiation energies and angles, to establish a matrix of calibration
2602 factors/coefficients or responses, or a calibration/response function. The second step, when
2603 used, applies these data to obtain the value of the desired quantity from the instrument
2604 indication.

2605 (192) All instrumentation should be fully tested prior to use. This procedure is especially
2606 important for applications in space where an exchange of instruments during a mission is
2607 often not possible. Failure of any part of the test should be clearly detailed and reasons for the
2608 failure considered. Fully tested means the determination of the instrument performance
2609 characteristics, including detection limit; tests of influence quantities, including other particle
2610 field components; and tests of the reliability of the complete system, including system
2611 software.

2612 (193) For a fully tested instrument, a reference calibration (determination of a single
2613 calibration factor or calibration coefficient for one set of reference conditions) is sufficient to
2614 ensure a traceable absolute dose measurement. The reference calibration of the instrument
2615 should be repeated at regular intervals. This may not be possible in space applications, but
2616 then there should be periodic checks on the performance which may be carried out using non-
2617 reference fields and a fixed procedure. In addition to the type test, some instrument should
2618 have a traceable individual normalization/calibration factor. For reusable dosimeters, this
2619 factor should be checked periodically and adjusted if necessary.

2620 (194) The response characteristics of all the types of devices should be determined by a
2621 combination of calculation and measurements. By Monte Carlo or other simulations the
2622 response of an instrument is determined in terms of particle fluence, and its energy
2623 distribution. The energy dependence of the response should be simulated for the particle
2624 types and range of energies relevant to the radiation fields in space, plus the angle
2625 dependence of response of the instrument, if any. The simulated instrument response must be
2626 benchmarked in reference radiation fields. For the simulation of the response characteristics
2627 of personal dosimeters, some irradiations should be performed with the dosimeter positioned
2628 on either an anthropomorphic phantom or a surrogate.

2629 (195) Where an instrument consists of more than one detector or more than one signal
2630 channel, the result of any algorithm to calculate the measured value is to be treated as the
2631 instrument indication in all determinations of calibration coefficient or factor or response.

2632 (196) Experimental response data for the active and passive devices used should be
2633 determined for the following energy ranges as appropriate: protons from below 10 MeV to 1
2634 GeV; helium from 10 MeV/u to 1 GeV/u; high-Z, high-energy ions (e.g. C, Si, Fe) from 50
2635 MeV/u to 1 GeV/u; electrons from 0.5 MeV to 10 MeV; and neutrons from about 1 MeV to
2636 400 MeV, monoenergetic or quasi-monoenergetic; plus response data for fields which
2637 replicate the field produced by the interactions of GCR with shielding material. All the
2638 radiation fields used must be well characterized and traceable to national standards (NMI) or
2639 related to NMIs via a traceability network (for example, quasi-monoenergetic neutron fields)
2640 or as part of an international comparison programme (for example, ICCHIBAN (Uchihori et

2641 al., 2002; Yasuda et al., 2006)).

2642 (197) There are a number of ISO standards for photon, beta and neutron radiation,
2643 covering radiation fields for most radiation protection situations on Earth. The situation for
2644 space applications is more complex. Well specified mixed radiation reference fields with very
2645 high energy particles and a large component of heavy ions do not exist on Earth. There are
2646 radiation fields available at very high energy accelerators (for example EG at GSI in
2647 Germany; HIMAC, TIARA, CYRIC, and RCNP in Japan; NASA Space Radiation Lab at
2648 BNL in the USA; TSL in Sweden; NPI in the Czech Republic; iThemba in South Africa;
2649 NFS in France, CERN in Switzerland) which include a broad range of secondary particles
2650 produced in the shielding surrounding a target. Such radiation fields replicate components of
2651 the radiation fields in space and have been used for testing and intercomparison
2652 measurements of various detector systems (Mitaroff et al., 2002). The precise specification of
2653 these radiation fields, however, can be a problem. Calculations are needed for the beam
2654 particle and secondary particles, fluence rate and its energy distribution, plus any scattered
2655 components. The specific detector response to heavy ions can also be determined using
2656 beams from heavy ion accelerators where the fluence rate of the heavy ion considered is well
2657 specified.

2658 (198) Determination of the dosimetric characteristics of the instrument (device or
2659 combination of devices) and its reference calibration are closely interlinked. The result of a
2660 response characterization is the detailed description of the dosimetric properties of a given
2661 instrument. This includes the dependence of the response on particle type, energy, angle of
2662 radiation incidence, and on various influence quantities. A reference calibration without a
2663 prior dosimetric characterization can be misleading, as the calibration can be misinterpreted
2664 as applying to the radiation field in space without correction. An instrument dedicated to a
2665 specific type of radiation (e.g. to neutrons) can have a response also for other particle types.
2666 This needs also to be taken into account by calibrations.

2667 (199) In modern instruments, the software has become of increasing importance for the
2668 generation of the measured value. Therefore, the final version of the software should be
2669 available at the beginning of the type test, as a great part of the software test is indirectly
2670 covered by the metrological test. The manufacturer should be aware of the fact that any
2671 change of the software may invalidate the type test. Dosimetry system software should be
2672 guided by the WELMEC software guide 7.2 (WELMEC, 2008).

2673

2674 **4.6 Accuracy and uncertainties for measurements in spacecraft**

2675

2676 (200) Astronauts are exposed to complex, multi-component fields that are difficult to
2677 determine routinely. Radiation quality-weighted organ absorbed doses can be significant, and
2678 a main objective is that the uncertainties in the fluence rate and its energy distribution and in
2679 the assessments of detector absorbed dose and radiation quality should be minimized.
2680 Nevertheless, one of the objectives should be to meet the general requirement that the total
2681 relative combined standard uncertainty of detector absorbed dose or dose equivalent should
2682 be minimized. The total relative uncertainty in a subsequent estimate of organ dose or dose
2683 equivalent will be greater.

2684 (201) The uncertainty in measurement assesses the measurement accuracy or metrological
2685 quality of measurements or fitness for purpose of a measured quantity value. Measurement
2686 accuracy is an assessment of the uncertainty. Guidance is based on documents and
2687 recommendations prepared by the Joint Committee for Guides in Metrology (JCGM), which
2688 gives definitions and guidance for metrology in general. These are available from the website

2689 of the Bureau International des Poids et Mesures: JCGM 200 (JCGM, 2008a), JCGM 100
2690 (JCGM, 2008b), and JCGM 104 (JCGM, 2009). These definitions and guidance are
2691 published by the International Organization for Standards (ISO) and the International
2692 Electrochemical Committee (IEC) (ISO/IEC Guide 99, 2007; ISO/IEC Guide 98-3, 2005;
2693 ISO/IEC Guide 98-1, 2009).

2694 (202) An essential aspect of quality assurance is assessing to what extent is it reasonable to
2695 believe that the reported number is a good estimate of the true dose value. The greater this
2696 belief, the confidence or probability that the measured value is within a certain defined range
2697 around the true value, or rather that the true value is within a certain range of the observed
2698 value, the better the quality of the measurement. In the evaluation of the uncertainty, all
2699 knowledge of the instrument and evaluating system both from experience and from type
2700 testing should be used possibly in combination with detailed information on the instrument
2701 assembly usage.

2702 (203) In order to obtain dose data of which the quality is traceable and can be recognized it
2703 is recommended that the terms and definitions given in the documents issued by the JCGM
2704 and the ISO framework should be followed. In the formulation stage, all input/influence
2705 quantities that may contribute to the uncertainty should be identified, and must be considered
2706 in the measurement model; all model input/influence quantities should be characterized by a
2707 best estimate and either a probability density function (PDF) or a (combined) standard
2708 uncertainty. The shape of the PDF can be taken from measured data or from an assigned
2709 distribution. Results from a type test or other characterization of the response of a dosimetry
2710 system may be used as inputs to the uncertainty assessment; other parameters such as
2711 standard uncertainty and coverage intervals must be derived from the PDF of the output
2712 quantity.

2713 (204) ICRP in Publication 75 (ICRP, 1997) recommends that "In practice, it is usually
2714 possible to achieve an accuracy of about 10 % at the 95 % confidence level for measurements
2715 of radiation fields in good laboratory conditions. In the workplace where the energy and
2716 directional distribution of the particles in the radiation field are generally not well known, the
2717 uncertainties of an assessment will be significantly greater. Non-uniformity and uncertain
2718 orientation of the radiation field will introduce errors in the use of standard models. The
2719 overall uncertainty at the 95% confidence level in the estimation of effective dose around the
2720 relevant dose limit may well be a factor of 1.5 in either direction for photons and may be
2721 substantially greater for neutrons of uncertain energy, and for electrons. Greater uncertainties
2722 are also inevitable at low levels of effective dose for all qualities of radiation." These
2723 statements of ICRP strictly apply to the assessment of E and H_T for occupational exposure at
2724 low doses on Earth. ICRU has published recommendations on the acceptable levels of total
2725 uncertainty for dose measurements in radiological protection in Reports 47 and 66 (ICRU,
2726 1992, 2001) which are broadly consistent with ICRP statements. ICRU recommends for
2727 single measurements of the operational quantities that "...in most cases, an overall
2728 uncertainty of one standard deviation of 30% should be acceptable" and states "The error of
2729 instruments may substantially exceed this limit at some radiation energies and for certain
2730 angles of incidence, but conform to it when they occur in a radiation field with a broad
2731 energy spectrum and broad angular distribution".

2732 (205) The statistical uncertainty of laboratory calibrations is commonly far less than the
2733 above uncertainties. However, the absorbed-dose and dose-equivalent response of devices are
2734 frequently appreciably energy- and angle-dependent. In order to minimize the total
2735 uncertainty in practical measurements, either determination of the response is required for the
2736 radiation field in which it is to be used, or a simulation of this field may be used to determine

2737 and apply correction factors to the calibration factor of an instrument. It may also be possible
2738 to calculate the response in this field from the knowledge of the field and of the detailed
2739 energy- and angle-dependence of response of the device. Frequently, it is the direction
2740 distribution of the field which has the largest influence. For measurements in space,
2741 additional information on the particle type, energy and direction distributions, are available.
2742 Using these data, accuracies of better than a factor of 1.5 at the 95% confidence level
2743 (equivalent to a standard deviation for a normal distribution of about 0.25) should be
2744 achieved for the estimation of organ absorbed dose and organ dose equivalent.

2745 (206) At doses approaching or exceeding mission risk limits, or career risk limits, the
2746 upper 95% coverage probability is used to include the measurement uncertainty for the
2747 assessment of the cancer risk projections from estimations of organ doses and radiation
2748 quality and tissue weighting factors (see Sect. 7.5). Obviously, reduction of measurement
2749 uncertainties is seen to be an important task for mission planning.

2750
2751

2752 **5. RADIATION FIELDS INSIDE SPACECRAFT AND ON** 2753 **PLANETARY SURFACES**

2754

2755 (207) The radiation field inside or near spacecraft includes various components of the
2756 primary radiation field in space (see Chapter 2) partially absorbed in the walls of the
2757 spacecraft and secondary radiation which is produced by scattering and reactions of the
2758 primary radiation in the walls and other materials within the spacecraft. Due to both the
2759 variation of the primary radiation field with time and to the non-homogeneous distribution of
2760 materials within the spacecraft the internal radiation field depends on the position in the
2761 spacecraft and on time during the mission.

2762 (208) The radiation field near planetary surfaces is determined by several factors, such as
2763 the existence and strength of a magnetic field, the thickness and composition of the
2764 atmosphere and the planetary material near to the surface. Important are the scattering,
2765 absorption and reactions in the planetary atmosphere and in the ground material.

2766

2767 **5.1 General**

2768

2769 (209) The physical description of the interaction of space radiation with matter requires
2770 knowledge of the energy and isotopic distribution of primary and secondary charge particles
2771 and neutrons produced in atomic and nuclear collision processes in the transport of radiation
2772 through matter. Computer codes describing proton, high-energy and charge (HZE) nuclei,
2773 and secondary radiation energy distributions and their transport through matter are used for
2774 shielding design of spacecraft and planetary habitats, and organ exposure assessments. The
2775 codes need to be benchmarked against space dosimetry results. The broad range of ion types
2776 and energies of the galactic cosmic radiation and solar particles, and the large number of
2777 materials of interest in spacecraft structures, planetary atmospheres, and tissues require a
2778 detailed description of the basic physical processes including the development of reliable
2779 computer models. Studies of potential risk mitigation including operational, shielding, and
2780 biomedical approaches must rely on theoretical models in the form of radiation transport
2781 codes to make projections and to support design studies of such mission.

2782 (210) Dominant physical processes in the penetration of high-energy nuclei through matter
2783 are energy loss through atomic and molecular collisions and the absorption and particle
2784 production from nuclear interactions with spacecraft materials and tissue. For heavy ions with

2785 high kinetic energies (> 100 MeV/u), nuclear absorption by fragmentation is the dominant
 2786 reaction mode (Hufner, 1985, Townsend et al., 1996). The nuclear absorption cross section
 2787 scales by the nuclear mass number to a power, $A^{1/3}$ and fragmentation of GCR nuclei is more
 2788 efficient per unit mass for materials with light constituent atoms. At lower energies (<100
 2789 MeV/u), elastic scattering, compound nucleus formation or excitations of discrete nuclear
 2790 levels that decay by gamma emission or particle emission are dominant interaction modes.
 2791 However, the short-range and large stopping powers of heavy ions reduce the importance of
 2792 nuclear reactions at lower energies except for neutrons. High-energy protons and neutrons
 2793 interact through knockout and spallation reactions (Hufner, 1985). Such processes lead to a
 2794 build-up of light particles ($Z \leq 2$) and the localized production near the primary track of
 2795 heavy ion target fragments with large values of LET and short ranges (Wilson, *et al.*, 1991,
 2796 Cucinotta *et al.*, 1996).

2797

2798 5.2 Physics of space radiation transport

2799

2800 5.2.1 Radiation transport

2801

2802 (211) The description of the passage of high-energy nuclei through matter can be made
 2803 using the Boltzmann transport equations that treat the atomic and nuclear collisions. The
 2804 equations may be solved by either numerical and analytic techniques - the straight ahead
 2805 approximation, or as an alternative, by Monte-Carlo techniques which sample from
 2806 interaction processes for individual primaries or their secondaries to develop histories of
 2807 charged particle passage and energy deposition in materials. The Monte Carlo method is a
 2808 widely used technique in particle physics and often applied in simulating radiation transport
 2809 through matter and the calculation of dose distributions.

2810 (212) The relevant Boltzmann transport equations are derived on the basis of conservation
 2811 principles (Wilson *et al.*, 2001) for the fluence rate $\phi_j(x, \Omega, E)$ of type j particles as:

$$2812 \quad \Omega \cdot \nabla \phi_j(x, \Omega, E) = \sum_k \iint \sigma_{jk}(\Omega, \Omega', E, E') \phi_k(x, \Omega', E') dE' d\Omega' - \sigma_j(E) \phi_j(x, \Omega, E) \quad (5.1)$$

2813 where $\sigma_j(E)$ and $\sigma_{jk}(\Omega, \Omega', E, E')$ are the media macroscopic cross sections. The $\sigma_{jk}(\Omega, \Omega', E, E')$
 2814 represent all those processes by which the particles of type k moving in direction Ω' with
 2815 energy E' produce a particle of type j in direction Ω with energy E . The fluence rate
 2816 $\phi_j(x, \Omega, E)$ is the main physical quantity used to determine the physical or biological response
 2817 by folding it with an appropriate response function for the physical or biological system
 2818 under study.

2819 (213) There may be several reactions, which produce a particular product, and the
 2820 appropriate cross sections for Equation (5.1) are the inclusive ones. The total cross section
 2821 $\sigma_j(E)$ with the medium for each particle type of energy E may be expanded as:

$$2822 \quad \sigma_j(E) = \sigma_j^{\text{at}}(E) + \sigma_j^{\text{el}}(E) + \sigma_j^{\text{f}}(E), \quad (5.2)$$

2823 where the first term refers to collision with atomic electrons, the second term is for elastic
 2824 nuclear scattering, and the third term describes nuclear reactions. The microscopic cross
 2825 sections and average energy transfer are ordered as follows:

$$2826 \quad \sigma_j^{\text{at}}(E) \sim 10^{-16} \text{ cm}^2 \quad \text{with } \delta E_{\text{at}} \sim 10^2 \text{ eV} \quad (5.3)$$

$$2827 \quad \sigma_j^{\text{el}}(E) \sim 10^{-19} \text{ cm}^2 \quad \text{with } \delta E_{\text{el}} \sim 10^6 \text{ eV} \quad (5.4)$$

$$2828 \quad \sigma_j^{\text{f}}(E) \sim 10^{-24} \text{ cm}^2 \quad \text{with } \delta E_{\text{r}} \sim 10^8 \text{ eV} . \quad (5.5)$$

2829 Over a distance of 1 g cm^{-2} of a material many atomic collisions ($\sim 10^6$) occur, many less
 2830 nuclear coulomb elastic collisions ($\sim 10^3$), while nuclear reactions are separated by up to
 2831 many cm depending on energy and particle type. For neutrons, it is $\sigma_n^{el}(E) \sim 0$ and the
 2832 nuclear elastic process appears as the first-order perturbation. Mean free paths for elastic
 2833 scattering of neutrons may become quite small, especially at low energies in the resonance
 2834 region (ICRU, 2000).

2835 (214) The solution of Equation (5.1) involves hundreds of multi-dimensional integro-
 2836 differential equations which are coupled together by thousands of cross terms and must be
 2837 solved self-consistently subject to boundary conditions that ultimately relate to the external
 2838 environment and the geometry of the astronaut's body and/or a complex spacecraft. A series
 2839 of approximate solutions can be studied and indicates a high level of accuracy for most
 2840 applications (Wilson *et al.*, 2001, Tweed *et al.*, 2005). The mean energy loss can be
 2841 introduced in a continuous slowing down approximation (csda), and straggling neglected for
 2842 the broad energy spectra of the space radiation. The highly directional coulomb cross section
 2843 for charged ions (Wong *et al.*, 1990) and nuclear elastic scattering for neutrons generally
 2844 dominate the second perturbation term. The angular dispersion and its effects on lateral beam
 2845 spread and range straggling are important corrections in comparing to laboratory
 2846 measurements. The nuclear elastic scattering is especially important to neutron fields and has
 2847 been treated using Monte Carlo or multi-group methods (Hughes, *et al.*, 1997). The third
 2848 perturbation term consists of complex energy and angle functions. Results from Monte Carlo
 2849 codes (Allsmiller *et al.*, 1965) provided the basis for the generation of analytical techniques
 2850 and the simplification of boundary conditions used in space shield code development (Wilson
 2851 *et al.*, 1991).

2852

2853 5.2.2 Atomic processes.

2854

2855 (215) The transport coefficients describe the atomic/molecular and nuclear processes by
 2856 which the particle fields are modified by the presence of material (Wilson *et al.*, 2001). As
 2857 such, basic atomic and nuclear theories provide the input to the transport code databases. The
 2858 first order physical perturbation on the right side of Equation (5.1) is the atomic/molecular
 2859 cross sections as noted in Equation (5.3) for which those terms in Equation (5.1) are
 2860 expanded about the energy moments l as

2861
$$S_n(E) = \sum_i \sum_l \epsilon_i^n \sigma_i(E) \text{ ,} \quad (5.6)$$

2862 where ϵ_i is based on the electronic excitation energy, and $\sigma_i(E)$ is the total atomic/molecular
 2863 cross section for delivering ϵ_i energy to the orbital electrons (including discrete and
 2864 continuum levels). The first moment ($n=1$) is the usual stopping power, and the usual
 2865 continuous slowing down approximation (csda) is achieved by neglecting the higher-energy
 2866 moments. The second moment represents the energy straggling (Payne, 1969) due to the
 2867 stochastic distribution of energy loss.

2868 (216) Stopping power data bases are derived semi-empirically as the Bethe reduction of
 2869 Equation (6) in terms of mean excitation energies and shell corrections (Fano, 1963, Wilson
 2870 *et al.*, 1991). The stopping power, S , is adequately described by the Bethe-Bloch formula for
 2871 most ion energies (Bichsel, 1992):

2872
$$S = \frac{4\pi Z_p^2 Z_T N_T e^4}{mv^2} \left\{ \ln\left(\frac{2mc^2 \beta^2 \gamma^2}{I}\right) - \beta^2 - \frac{C(\beta)}{Z_T} + Z_p L_1(\beta) + Z_p^2 L_2(\beta) + L_3(\beta) \right\} \quad (5.7)$$

2873 where e is the electronic charge, N_T is the density of target atoms, m is the mass of the
 2874 electron, c is the speed of light, $\beta=v/c$, and I is the mean excitation energy. In Equation (5.7),
 2875 the various terms are the shell correction $C(\beta)$, Barkas correction, $L_1(\beta)$, Bloch term, $L_2(\beta)$,
 2876 and Mott and density corrections $L_1(\beta)$. The range of the ion is evaluated from the stopping
 2877 power as

2878
$$R(E) = \int_0^E \frac{dE'}{S(E')} \quad (5.8)$$

2879 (217) The second energy moment is related to energy or range straggling and provides
 2880 corrections to the ion slowing down spectrum (Fano, 1963, Payne, 1969). For broad-energy
 2881 beams conditions of GCR or SPEs transport straggling effects are negligible, however they
 2882 are important for laboratory studies with mono-energetic beams and for understanding
 2883 radiation detector response. The next physical perturbation term is the Coulomb scattering by
 2884 the atomic nucleus and is typically represented by Rutherford scattering modified by
 2885 screening of the nuclear charge by the orbital electrons using the Thomas-Fermi distribution
 2886 for the atomic orbits. The total nuclear Coulomb cross section found by integrating over the
 2887 scattering directions is related to the radiation length. The differential cross section is highly
 2888 peaked in the forward direction, and only after many scatterings is significant beam
 2889 divergence seen. Numerical solutions to the Coulomb multiple-scattering problem have been
 2890 investigated for many years (Fermi, 1940) and accurately describe experimental data with
 2891 HZE (Wong *et al.*, 1990) or proton beams (Carlsson and Rosander, 1973).

2892
 2893 **5.2.3 Nuclear interactions**
 2894

2895 (218) The extent of the nuclear interaction cross section database required for the transport
 2896 of cosmic radiation is for energies from 1 MeV/u to energies of tens of GeV/u, including a
 2897 large number of projectile and target material combinations. The types of cross sections
 2898 required for transport involve total yields and multiplicities and inclusive secondary energy
 2899 spectra, inclusive double differential cross sections in angle and energy. The total absorption
 2900 plays a critical role in ensuring a reasonable solution to the Boltzmann equation including the
 2901 accuracy of particle conservation as a function of depth in the shield (Wilson *et al.*, 1991).
 2902 Similarly in Monte-Carlo approaches the absorption cross section plays the critical role of
 2903 determining the probability of interaction events along the trajectory of a primary particle in
 2904 the shielding. In addition, exclusive cross sections are used in some event generators in
 2905 Monte-Carlo transport codes. The total cross section σ_{TOT} is found from the elastic amplitude
 2906 in the forward direction as found in the optical theorem (Wilson *et al.*, 1991) as the imaginary
 2907 part of the elastic scattering amplitude (Im f):

2908
$$\sigma_{TOT} = \frac{4\pi}{k} \text{Im} f(q=0) \quad (5.9)$$

2909 where q is the momentum transfer, and k the relative momentum of projectile and target
 2910 nuclei.

2911 (219) The total absorption (ABS) cross section is then also found from the elastic
 2912 scattering amplitude by using:

2913
$$\sigma_{TOT} = \sigma_{ABS} + \sigma_{EL} \quad (5.10)$$

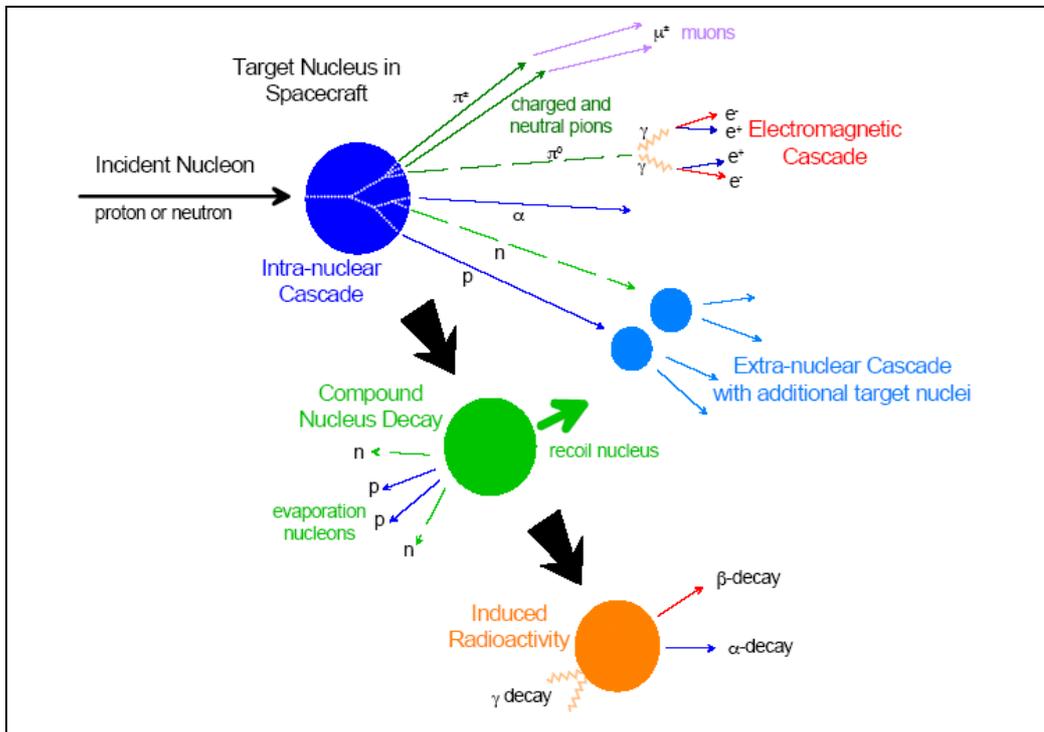
2914 where σ_{EL} is the total elastic cross section. Formula for these cross sections can be derived
 2915 from microscopic theories of nuclear multiple scattering (Cucinotta *et al.*, 1997). The
 2916 absorption cross sections are accurately represented by energy dependent variants of the
 2917 Bradt-Peters equation (Townsend *et al.*, 1986a)

2918
$$\sigma_{\text{ABS}} = \pi r_0^2 c_1(E)(A_p^{1/3} + A_T^{1/3} - c_2(E))^2 \tag{5.11}$$

2919 where r_0 , $c_1(E)$ and $c_2(E)$ are parameters fit to experimental data. Absorption cross sections
 2920 have been well studied both experimentally and theoretically and are known with a few
 2921 percent uncertainties (Tripathi, 2001). The absorption cross section rises at low energy as
 2922 reaction channels open and reaches a minimum at a few hundred MeV/u before rising again
 2923 as meson production channels open.

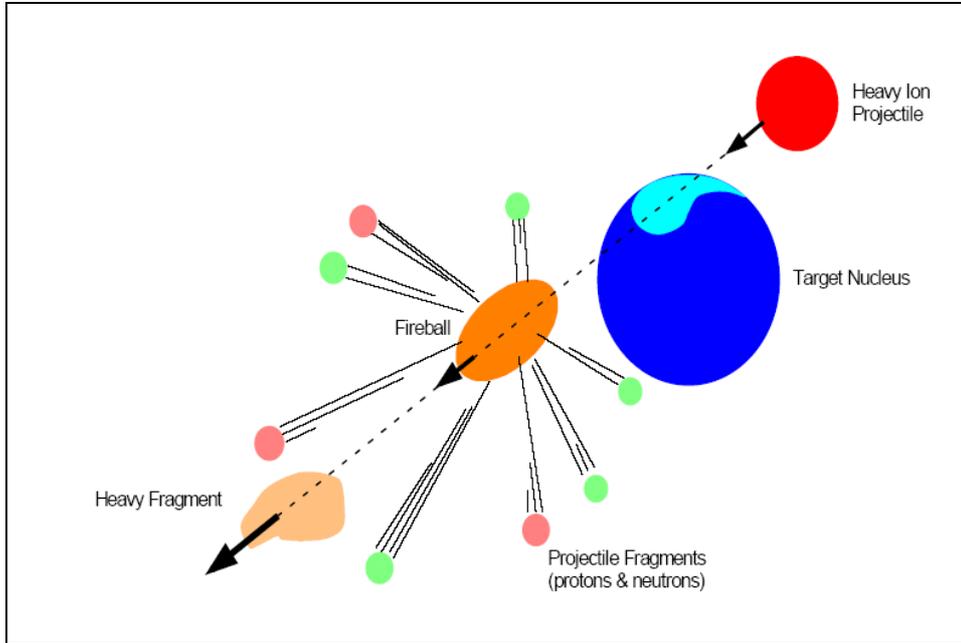
2924 (220) Table 5.1 shows reaction partners and secondaries of relevant reactions broken into
 2925 distinct reaction types or mechanisms. Low energy evaporation products including heavy ion
 2926 target fragments are high-LET events important in biological damage. Knockout products
 2927 from proton or neutron reactions and projectile fragments from nuclei of GCR are typically
 2928 of low to moderate LET, however, their large ranges leads to radiation build-up through
 2929 further reactions. The abrasion-ablation models (Hufner *et al.*, 1975, Townsend *et al.*, 1986b,
 2930 1996, Wilson *et al.*, 1995b, Cucinotta, et al., 1997) are used to describe heavy ion
 2931 fragmentation cross-sections. The description of nuclear reactions through abrasion (particle
 2932 removal during ion-ion interaction) and ablation (nuclear de-excitation after the abrasion
 2933 step) is illustrated in Fig. 5.1, which shows the roles of projectile overlap, fireball formation
 2934 in central regions, and the decay of the pre-fragment spectators (NCRP, 2006). The
 2935 individual steps of abrasion and ablation can be described in both semi-classical or quantum
 2936 mechanical approaches (Cucinotta *et al.*, 1995; 2007). These different reaction processes
 2937 have been described by quantum multiple scattering theories (QMST), semi-classical
 2938 methods such as quantum molecular dynamics, or Monte-Carlo approaches to nuclear
 2939 reactions using an intra-nuclear cascade model (see Durante and Cucinotta, 2011 and
 2940 references therein for a review of nuclear reaction models).

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2946 Fig. 5.1a. Schematic diagram of the reaction of an incident nucleon with a target nucleus at high
 2947 energies (ICRU, 1978).
 2948
 2949



2950
 2951 Fig. 5.1b. Schematic diagram of a relativistic heavy ion reaction with a target nucleus (Miller, 1997).
 2952
 2953

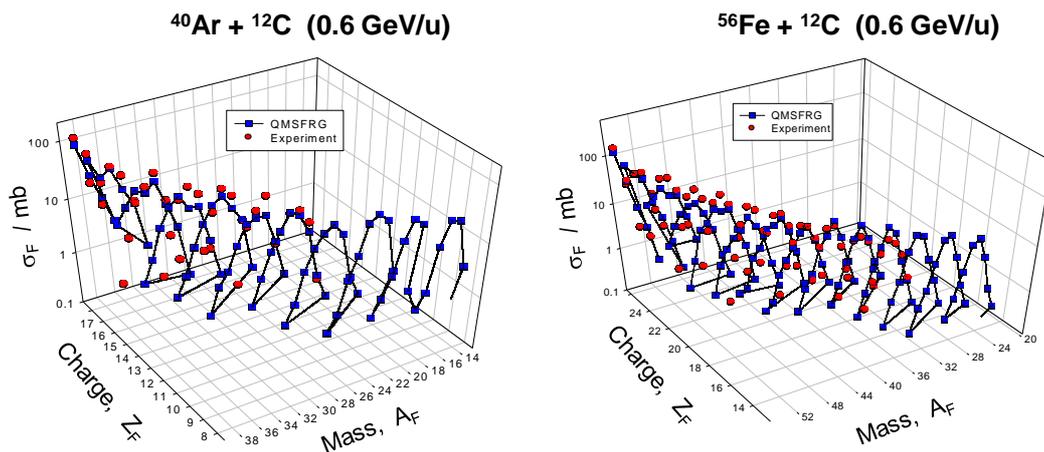
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 2955

Table 5.1. Reaction products in nuclear reactions important to space radiation studies.

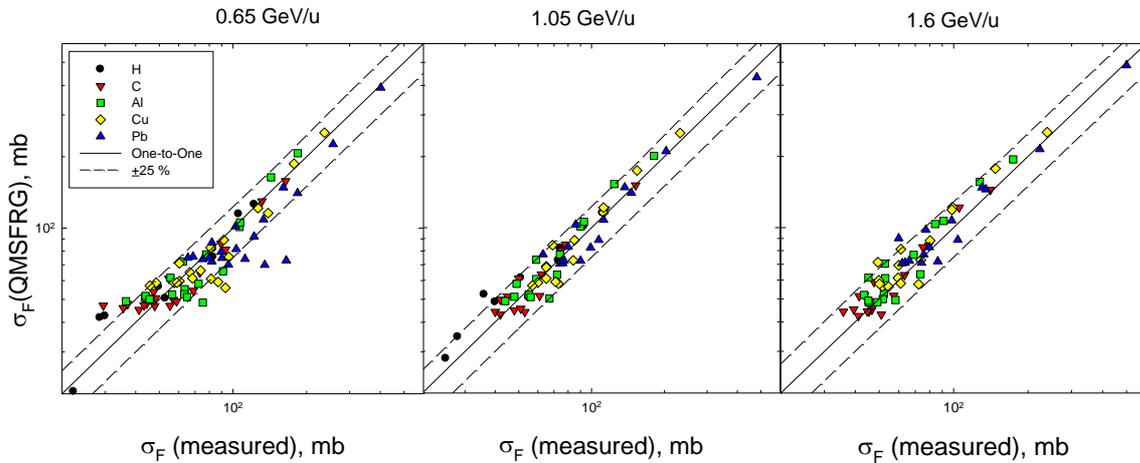
REACTION TYPE	SECONDARY	MECHANISM	COMMENT
Nucleon-Nucleus	Nucleon	Evaporation Knockout and elastic, quasi-elastic scattering	High LET, small range Large range
Nucleon-Nucleus	Light particle (d,t,h, α)	Evaporation Knockout, pickup	High LET, small range Large range
Nucleon-Nucleus	Heavy recoil	Elastic scattering Fragmentation or spallation	High LET, small range High LET, small range
Nucleus-Nucleus	Nucleon or light particle	Target or projectile Knockout or evaporation	Small LET, large range High LET, small range
Nucleus-Nucleus	Heavy ion	Projectile Fragment	Mod. LET, large range
Nucleus-Nucleus	Heavy ion	Target Fragment	High LET, small range
Nucleon or Nucleus-Nucleus	Pion, Kaon, anti- nucleon, gamma	Target reaction Proj. $E > 500$ MeV/u	Deep penetration (> 50 g/cm ²)

2956
 2957 (221) Examples of fragmentation cross sections for Ar and Fe projectiles are shown in Fig.

2958 5.2 in comparison to the quantum multiple scattering fragmentation (QMSFRG) model
 2959 (Cucinotta, et al, 2002, 2006b, and 2007). Available fragmentation cross section data sets for
 2960 target atoms of interest for space missions were reviewed by Durante and Cucinotta (2011)
 2961 and NCRP (2006). One feature of the elemental distribution of the fragments is the strong
 2962 even-odd effect in the charge number of the fragments observed. The effect appears to be
 2963 largest for intermediate mass projectiles ($A=20$ to 40) and depends on the isospin of the
 2964 projectile (Knott et al., 1996, 1997). Theoretical models provide a good representation of the
 2965 odd-even effect if accurate nuclear de-excitation models are used (Cucinotta *et al.*, 2002,
 2966 2006b, 2007). Figure 5.2 shows comparisons of fragmentation production cross sections of
 2967 the QMSFRG model with experiments for several beam energies of ^{56}Fe interacting with a
 2968 variety of target nuclei. Good agreement between experiments and the QMSFRG model
 2969 within 25% for most fragment cross sections has been achieved as shown in Fig. 5.3. A larger
 2970 data base for cross sections for proton and neutron induced reactions now exists with recent
 2971 surveys of such data provided in ICRU Report 63 (ICRU, 2000). Above kinetics energies of a
 2972 few 1000 MeV/u, multiple meson production processes are dominant and reactions models
 2973 based on patron or quark-gluon models can be used to describe nuclear interaction cross
 2974 sections (Fasso *et al.*, 2005).



2975 Fig. 5.2. Comparisons of the QMSFRG model (Cucinotta *et al.*, 2006b) to experiment for isotopic
 2976 distribution of fragments for ^{40}Ar on ^{12}C at 0.6 GeV/u (left panel) and ^{56}Fe on ^{12}C interactions at 0.6
 2977 GeV/u (right panel).
 2978
 2979



2980

2981 Fig. 5.3. Comparisons between experiments and the QMSFRG model for fragmentation cross
 2982 sections for ⁵⁶Fe projectiles of different energies interacting with several target nuclei (Cucinotta et
 2983 al., 2006b) where the dashed lines show ±25 % ranges of the experimental data.

2984

2985 (222) The momentum distribution of heavy projectile fragments is forward peaked and
 2986 described as a Gaussian distribution in the projectile rest frame with a small downshift in the
 2987 average momentum from the projectiles velocity. The longitudinal momentum width, σ_L , is
 2988 well described by (Goldhaber, 1974):

2989

$$\sigma_L = \sigma_0 \left[\frac{n(A_p - n)}{A_p - 1} \right]^{1/2} \quad (5.13)$$

2990 where n is the number of nucleons removed from the projectile and σ_0 is approximately
 2991 related to the Fermi-momentum of the projectile, p_F , by $\sigma_0 = p_F/\sqrt{5}$. The transverse width is
 2992 approximately the same as the longitudinal for heavier fragments. A small momentum
 2993 downshift of the fragments relative to the projectile also occurs and is dependent on the
 2994 fragment mass (Tull, 1990). Transformation of the Gaussian distribution to the laboratory rest
 2995 frame reveals a narrow angular distribution for the projectile fragments that are strictly
 2996 forward peaked in a narrow cone ($<5^\circ$), which leads to the success of the straight-ahead
 2997 approximation in transport models. For lighter fragments the longitudinal and transverse widths
 2998 diverge and the Gaussian model breaks down. This is due to the multiple sources for light
 2999 particle production including projectile abrasion, projectile ablation, target abrasion, and
 3000 target ablation, as well as a possible intermediate source due to the formation of an
 3001 intermediate rapidity fireball in central collisions.

3002

3003 5.3 Proton, neutron and heavy ion transport codes

3004

3005 (223) Several radiation transport codes have been developed for applications in
 3006 radiotherapy, physical experiments at high-energy accelerators, detector simulations, and
 3007 radiation protection in space. The different radiation transport codes utilize distinct nuclear
 3008 data bases and methods including their treatment of nuclear interactions, secondary radiation,
 3009 and shielding geometries. It is unlikely that space radiation problems can be handled with a
 3010 one-size fits all approach and the specific application will drive the method to be used.
 3011 Complex spacecraft and organ geometry are described using ray-tracing distributions,
 3012 combinatorial geometry models of complex structures, or voxel based methods. Ray tracing
 3013 methods are able to treat thousands of spacecraft parts accurately and can directly integrate

3014 engineering designs in a CAD format. Ray tracing methods have only been developed for bi-
3015 directional (forward-backward scattering); however they have been shown to be quite
3016 accurate for the omni-directional radiation fields in space. Combinatorial geometry models,
3017 often used by Monte-Carlo codes, rely on approximations that paradoxically wash out the
3018 fine details of surface and angular effects that 3D-transport models are intended to describe.

3019 (224) Models of the GCR, trapped radiation or SPE's are usually used as the boundary
3020 condition for transport codes. Several transport codes used in space applications are briefly
3021 summarized here:

3022

3023 **FLUKA** (FLUctuating KAscades)

3024 (225) FLUKA is a general purpose Monte Carlo program for calculations of particle and
3025 photon transport (Fassò et al., 2005; Battistoni et al., 2006) which can simulate the
3026 interactions and propagation in matter of approximately 60 different particles, including
3027 heavy ions (<http://www.fluka.org>). The program can also describe the transport of polarised
3028 photons (e.g., synchrotron radiation) and optical photons. Photonuclear interactions can be
3029 simulated. Time evolution of the radioactive nuclei inventory and tracking of emitted
3030 radiation from unstable residual nuclei can be performed.

3031 (226) Depending on the energies of the primary particles, hadronic interactions are
3032 simulated by different physical models. For higher energies, the Dual Parton Model is used.
3033 Below 3-5 GeV/c the PEANUT package includes a very detailed Generalised Intra-Nuclear
3034 Cascade (GINC) and a pre-equilibrium stage, while at high energies the Gribov-Glauber
3035 multiple collision mechanism is included in a less refined GINC. Nuclear interactions
3036 generated by ions are treated through interfaces to external event generators, except for the
3037 low energy (less than 150 MeV/u) range, for which a model based on the Boltzmann Master
3038 Equation (BME) has been implemented. The RQMD (Relativistic Quantum Molecular
3039 Dynamics) generator is invoked from 100 MeV/u to 5 GeV/u, and the DPMJET code is used
3040 for energies over 5 GeV/u.

3041 (227) The transport of charged particles is described by applying a multiple scattering
3042 algorithm based on Moliere's theory of Coulomb scattering. The algorithm includes an
3043 accurate treatment of curved trajectories in magnetic fields. The energy loss is determined
3044 according to the Bethe-Bloch theory and from bremsstrahlung and pair production. Ionisation
3045 fluctuations are accounted for.

3046 (228) For neutrons with energies lower than 20 MeV, FLUKA employs a multi-group
3047 transport algorithm, which uses a subdivision of the neutron energy range in 260 groups and
3048 is based on neutron cross section libraries containing more than 200 different materials,
3049 selected for their use in physics, dosimetry and accelerator engineering. Energy depositions
3050 for nuclei other than hydrogen are calculated by kerma coefficients.

3051 (229) FLUKA can handle very complex geometries, using an improved version of the
3052 well-known Combinatorial Geometry (CG) package. Repetitive structures (lattices) and voxel
3053 geometries can be handled. Various visualisation and debugging tools are also available.

3054

3055 **GEANT4**

3056 (230) The Monte-Carlo code Geant4 (Agostinelli, 2003 and Allison, 2006) states for
3057 GEometry ANd Tracking and is a software toolkit for simulating the passage of particles
3058 through matter. It has been developed and maintained by the Geant4 Collaboration which is a
3059 worldwide teamwork of physicists and software engineers (see:
3060 <http://geant4.web.cern.ch/geant4>). Geant4 and its predecessors were designed to utilise the

3061 physics models, to handle complex geometries, and to enable its easy adaptation for optimal
3062 use in different sets of tasks. It has its applications in high energy, nuclear and accelerator
3063 physics as well as studies in radiation protection, medical and space science.

3064 (231) The toolkit includes facilities for handling tracking, geometry, physics models,
3065 detector response, run management, visualisation and user interface. The software offers a
3066 large set of physical processes (e.g. electromagnetic, hadronic and optical model), different
3067 type of particles (leptons, bosons, mesons, baryons, etc) and databases with properties of
3068 matter and elements. Physics processes cover a wide range of energies which spans from 250
3069 eV up to TeV depending on the case. Several of the modules contained in Geant4 are re-
3070 creations of physics in other codes including e. g. HZETRN, HETC. Spacecraft geometry
3071 models are available including multi-layered shielding simulation Software (MULASSIS)
3072 (Bernabeu and Casanova, 2007).

3073 (232) The toolkit is implemented in C++ programming language making use of object
3074 oriented technique. This approach allows users effectively to manage complexity and limit
3075 dependencies by defining a uniform interfaces and common organisational principles in order
3076 to create an application for solving a specific problem.

3077

3078 **HETC-HEDS** (High Energy Transport Code-Human Exploration)

3079 (233) HETC is a high energy Monte Carlo radiation transport code and was developed at
3080 Oak Ridge National Laboratory (Townsend et al., 2002). The code was originally developed
3081 for transport calculations with incident high-energy protons, neutrons, π^+ , π^- , π^+ or π^- only.
3082 Lateron Townsend et al. (Townsend et al., 2005) have extended the model especially for
3083 space radiation shielding applications. The code, now called HETC-HEDS, has been
3084 modified to include the transport of heavier nuclei (Charara et al., 2008).

3085 (234) HETC-HEDS includes nucleus-nucleus cross sections, range-energy tables scaled
3086 from the proton data, and a nuclear collision module for heavy ion interactions. Nonelastic
3087 nucleon collisions and charged-pion collisions with hydrogen at energies above 3.5 GeV and
3088 2.5 GeV, respectively, are treated by using calculational methods that utilize experimental
3089 data for the total non-elastic n-p, p-p, π^+ -p and π^- -p cross sections and analytic fits to
3090 experimental data. The intra-nuclear-cascade evaporation concept of particle-nucleus
3091 interaction is used to determine the effect of particle-nucleus collisions below 3.5 GeV for
3092 nucleons and 2.5 GeV for charged pions. Following the intranuclear-cascade, the excitation
3093 energy left in the nucleus is treated using an evaporation model. The particles allowed during
3094 evaporation include protons, neutrons, d, ^3H , ^3He , and ^4He .

3095 (235) HETC-HEDS used the combinatorial geometry package so virtually arbitrary
3096 geometries are allowed. Each particle in the cascade is followed until it eventually disappears
3097 by escaping from the geometric boundaries of the system, undergoes nuclear collision or
3098 absorption, comes to rest due to energy losses from ionisation and excitation of atomic
3099 electrons, or decays in the case of pions and muons.

3100

3101 **HZETRN** (High charge (Z) and Energy Transport code)

3102 (236) HZETRN was developed by Wilson et al. (1991, 2004) at NASA. It solves the
3103 Boltzmann equation using numerical methods. The early version used the straight-ahead
3104 approximation but more recent versions use bi-directional transport or 3D transport for
3105 laboratory studies (Wilson, et al., 2009). Recently ray tracing representations of voxel models
3106 of human geometries have been developed (Slaba et al., 2009). Nuclear interactions are
3107 treated by the NUCFRG2 Model (Wilson et al., 1994a) or QMSFRG model (Cucinotta *et al.*,

3108 2007) with proton and neutron cross sections described by the Bertini and Ranft models. To
3109 treat complex radiation geometries ray tracing methods are utilized, which is powerful
3110 because spacecraft engineers can utilize their designs directly in the transport code
3111 evaluations. Computer runs on small computer workstations can be performed in less than
3112 one hour for complex spacecraft geometries with multi-layer materials and deep shielding
3113 conditions ($>100 \text{ g/cm}^2$) combined with models of the organ shielding of the human body.

3114
3115 **MCNPX** (Monte-Carlo N-Particle eXtended)

3116 (237) The Los Alamos Monte-Carlo code MCNPX (see: <http://mcnpx.lanl.gov>) and its
3117 predecessors are very widely distributed (Waters, 2002; Pelowitz, 2008). The code is capable
3118 of tracking many particle types (nucleons and light ions) and over 2000 heavy ions up to very
3119 high energies. It uses standard evaluated data libraries for neutrons, photons, electrons,
3120 protons and photonuclear interactions and uses physics models for other particle types and at
3121 energies for which tabular data are not available.

3122 (238) Current physics modules include the Bertini and Isabel models taken from the
3123 LAHET Code system, CEM 03, and INCL4 (James et al., 2009). The incorporation of a
3124 heavy ion physics model has enabled the transport of recoil nuclei. This model automatically
3125 transports all residuals that are produced from any reaction even if the source particle is not a
3126 heavy ion. Current stopping powers for heavy ions have been adjusted in an *ad hoc* fashion
3127 (Pelowitz, 2008) so that they better match SRIM results (Ziegler et al., 2008). Charged
3128 particles are slowed down to a lower total energy limit of 5 MeV, at which point their
3129 remaining energy is locally deposited.

3130
3131 **PHITS** (Particle and Heavy-Ion Transport code System)

3132 (239) The Monte-Carlo radiation transport code PHITS (see: <http://phits.jaea.go.jp>) was
3133 developed under collaboration with several institutes including JAEA, RIST, KEK and
3134 Chalmers University of Technology (Niita et al., 2010; Sihver et al., 2010a). Nuclear
3135 interactions are described by various models such as JAM and JQMD up to 100 GeV/u. The
3136 code can determine the energy of charged particles emitted from low-energy neutron-induced
3137 nuclear reactions, using the event generator mode (Iwamoto et al., 2007, Niita et al., 2007) in
3138 combination with nuclear data libraries. This feature enables the direct calculation of dose
3139 equivalent in organs or tissues which cannot be calculated by employing the conventional
3140 kerma approximation. The accuracy of the code for use in space dosimetry was well verified
3141 by calculating neutron spectra inside Space Shuttle (Sato et al., 2006) and doses inside
3142 anthropomorphic phantoms (Sato et al., 2011), using simplified geometries of spacecraft. The
3143 code is also used for the computational analyses of the MATROSHKA and MATROSHKA-
3144 R experiments in space (Sihver et al., 2010b and Koliskova et al., 2012).

3145
3146 **5.3.1 Inter-comparison and validation of radiation transport codes**
3147

3148 (240) An assessment of the accuracy of space radiation transport models for prediction of
3149 energy spectra of charged particles and neutrons after primary radiation has passed matter can
3150 be made by comparisons to laboratory experiments with proton and heavy ion beams or from
3151 measurements in spacecraft. Spaceflight measurements, however, involve many factors such
3152 that potential inadequacies in radiation transport models are difficult to isolate relative to
3153 possible inaccuracies in environmental or shielding models. Also, space validation is limited
3154 by the access to space and current spacecraft materials, and may not be representative of

3155 model predictions for other material types such as those that occur on planetary surfaces or in
3156 advanced materials selection concepts. In this respect, laboratory validation is advantageous
3157 to validate radiation transport computer codes and associated data base models and to provide
3158 tests for studying material properties for reducing biological doses (Schimmerling *et al.*,
3159 1999). Nevertheless, spaceflight measurements provide important tests of predictive
3160 capability of several factors and are needed for final validation of transport codes.

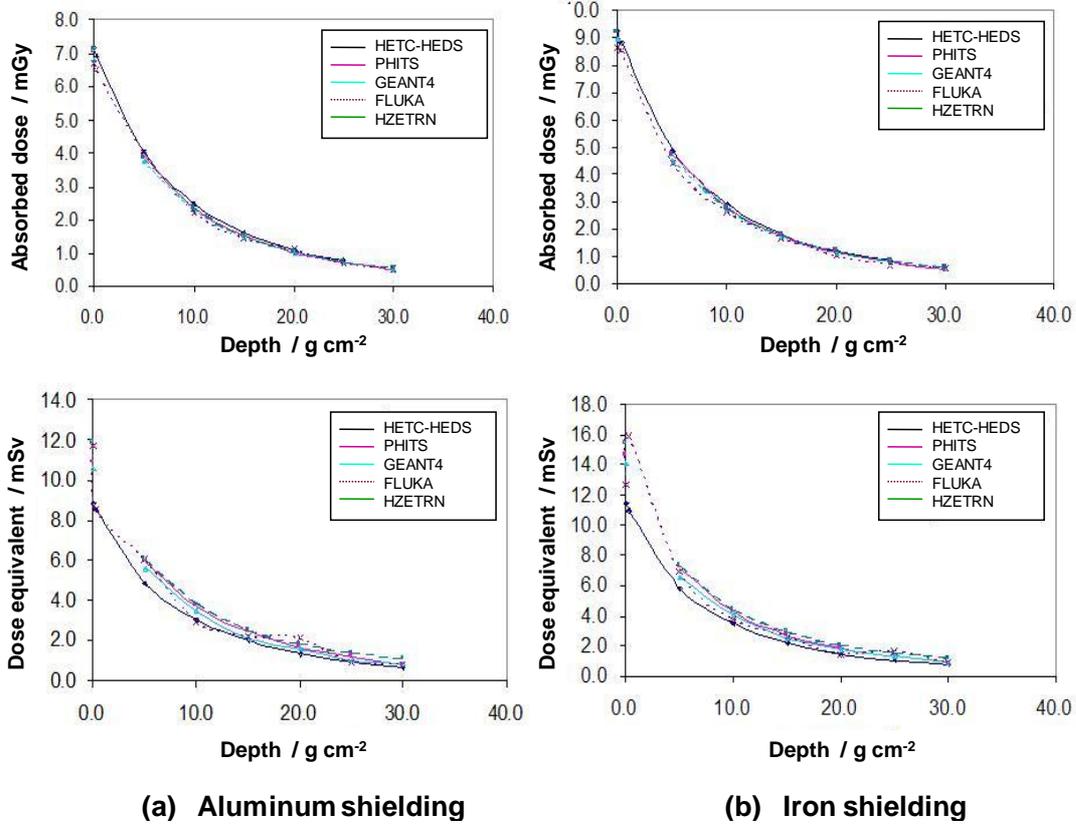
3161 (241) Recently an extensive benchmarking of the calculated projectile fragmentation
3162 cross-sections from the reactions of 300 - 1000 MeV/u Si-28, Ar-40, and Fe-56 ions on
3163 polyethylene, carbon, aluminum, and copper targets (relevant to space radiation protection)
3164 has been carried out using PHITS, FLUKA, HETC-HEDS, and MCNPX. Calculated results
3165 were compared with measurements (Sihver *et al.*, 2008).

3166 (242) An over-all “reasonable” agreement between calculations and measurements was
3167 found. However, a general trend of a slight underestimation of the calculated fragment
3168 production cross-sections (partial charge-changing cross-sections) has been observed. The
3169 code HETC-HEDS seems to underestimate these cross-sections more than the other codes
3170 included in this benchmarking. PHITS also seems to underestimate the total charge-changing
3171 cross-sections, which is in agreement with other observations (Sihver *et al.*, 2007).

3172 (243) A recent inter-comparison of transport codes for SPE and GCR test cases indicates
3173 fairly good agreement between the various codes (Wilson *et al.*, 2009; Heinbockel *et al.*,
3174 2011). Fig. 5.4 shows comparisons of results of depth-dose distributions from different codes
3175 for a large SPE. Fig. 5.5 shows a comparison of energy spectra of secondary particles
3176 produced by protons and Helium ions of GCR at solar minimum calculated with various
3177 codes.

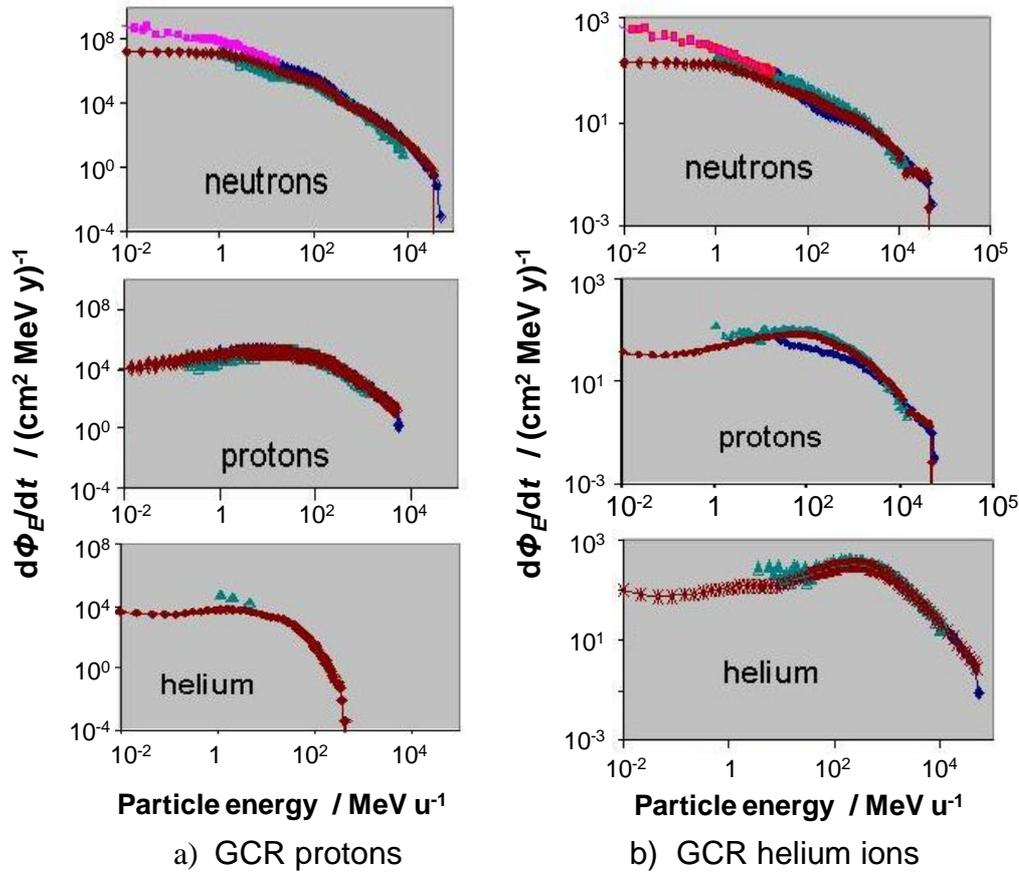
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Fig. 5.4. Inter-comparison of transport codes for depth-dose distributions of absorbed dose and dose equivalent from solar particle events in aluminum and iron shielding (Wilson et al., 2009).



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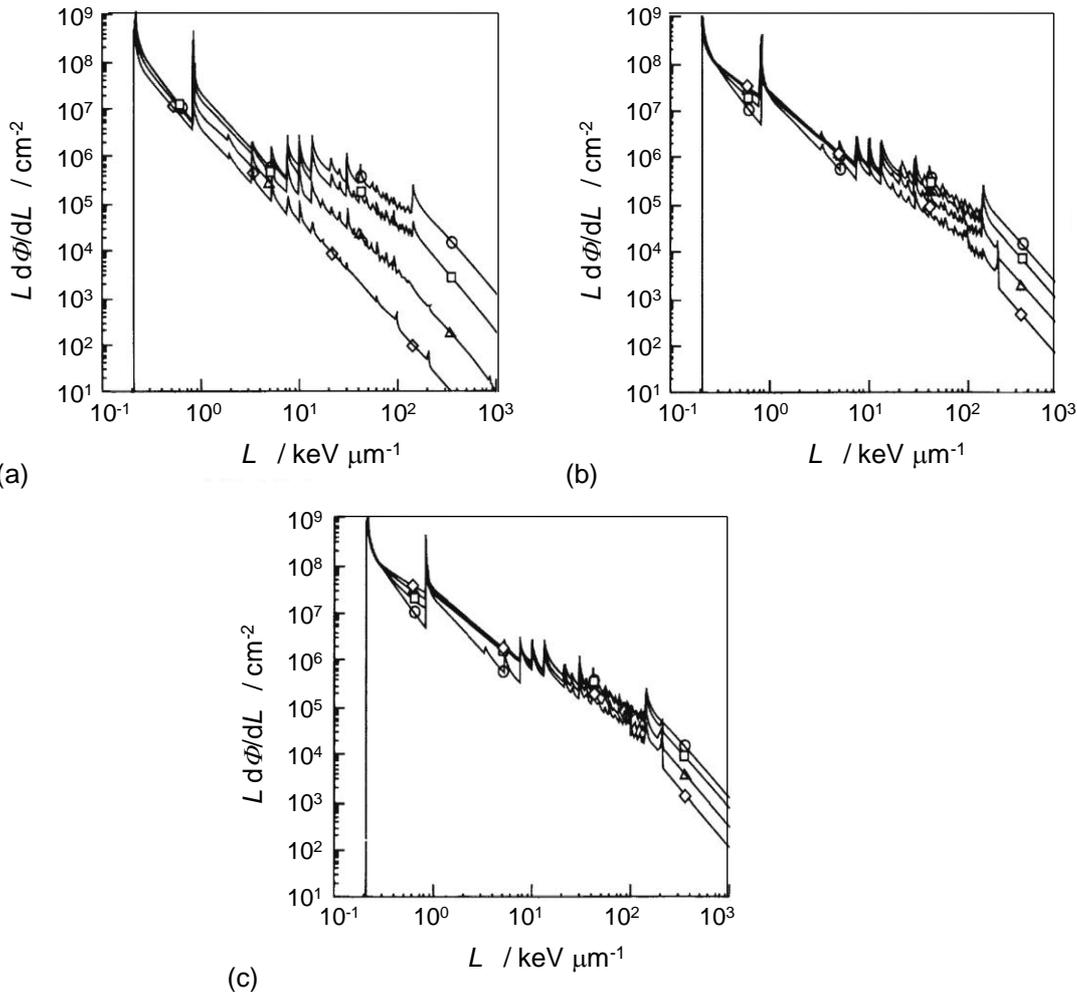
Fig. 5.5. Inter-comparison of energy spectra (spectral fluence rate, $d\Phi_E/dt$) of light ions from GCR at solar minima calculated by different transport codes (Wilson et al, 2009).

◆ HETC-HEDS, ▲ Fluka, ◆, * HZETAN2006

5.4 Radiation Fields inside spacecraft

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(244) Radiation fields inside spacecraft are determined by the external radiation incident on the spacecraft and the secondary radiation produced by the interactions with the walls and the equipment inside and outside the spacecraft. The internal radiation field varies with time due to the variation of the external radiation (see Chapter 2) and with the location in the spacecraft due to the specific arrangement of the equipment and the shielding properties of the different walls and spacecraft components. Important effects are absorption, scattering, and degradation of the primary radiation, and production of secondary radiation. Hence the radiation field inside a spacecraft –and also in the outer region near to the spacecraft – includes projectile and target fragment, neutrons, photons, pions and muons in addition to the radiation components of the primary field. Assessment of the inner radiation field and its components can be performed either by measurements or by simulations using environmental models and radiation transport codes.



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Fig. 5.6. LET distribution of particle fluence in space calculated using the HZETRN code for increasing amounts of liquid hydrogen, water, and aluminum shielding (Wilson et al., 1995c).

○ without shielding; with shielding thickness of □ 5 g cm⁻², Δ 15 g cm⁻², ◇ 30 g cm⁻².

(245) The exterior environment is modified by the types and amounts of radiation shielding. The LET distributions of particle fluence for increasing amounts of liquid hydrogen, water or aluminum shielding are shown in Fig. 5.6. The higher fluence at large values of LET (>50 keV/μm) for aluminum compared to the other materials is due to the contributions of secondary neutrons and charged particles produced in the shielding, which is reduced for materials containing hydrogen.

(246) The measurements on NASA space shuttle (STS) flights over many years and on the Russian space station MIR have allowed for a large number of comparisons of radiation transport calculations to flight measurements (Cucinotta et al., 2000b). Passive measurements with nuclear etched track detectors (PADC) have limitations at both low LET (< 5 keV/μm) tracks and short-tracks from target fragments or stopping GCR ions of high-LET tracks. The use of active dosimeters on STS flights has allowed for separation of GCR contributions from that of trapped protons which is not possible with passive dosimetry (Badhwar and Cucinotta, 2000). Active detector measurements include tissue equivalent proportional counters, charged particle telescopes (Badhwar et al., 1995), and active Bonner spheres embedded with proportional counters (Koshiishi et al., 2007).

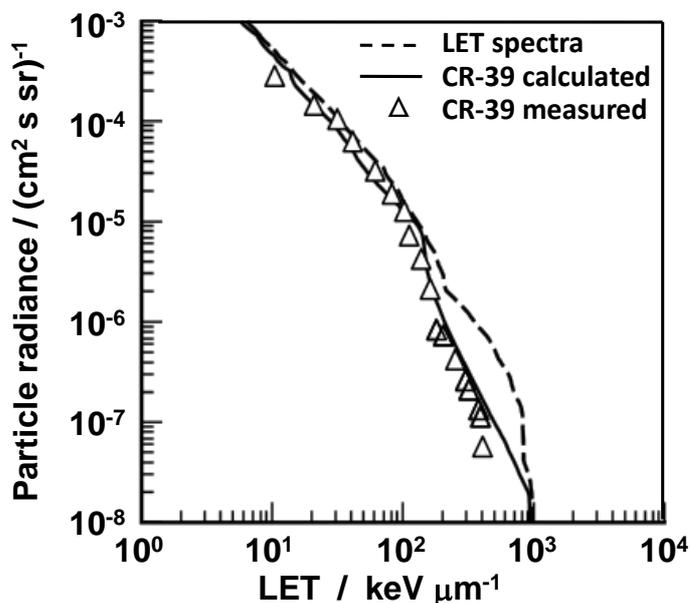


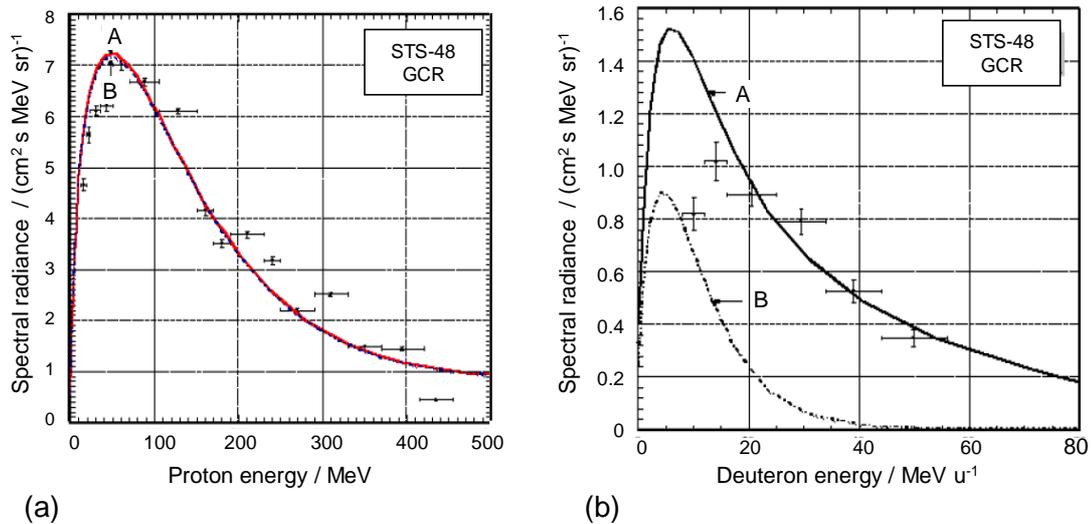
Fig. 5.7. Comparisons of LET distributions of particle radiance measured by nuclear etched track detectors (CR-39) to LET distributions and model calculations of LET distributions on Space shuttle mission (Wilson et al., 1994b, Shinn et al., 1998).

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(247) Measurements on STS flights were performed by Badhwar et al. (2000) using a cylindrical tissue equivalent proportional counter (TEPC) with a length-to-diameter ratio of 1 simulating a 2 μm diameter site and covering a lineal energy range of 0.25 keV/ μm to 1250 keV/ μm . TEPC measurements on several missions in space have been compared to calculations using HZETRN for total dose and dose equivalent. The comparisons use the free space GCR model of Badhwar and O'Neill (1992) and representations of the STS or Mir shielding distribution around the detectors. The difference is found to be less than 15% for the majority of the comparisons. A comparison of LET distributions measured by PADC and calculated using the HZETRN code is shown in Fig. 5.7. They show a good agreement when the response of the PADC to short tracks is additionally considered (Wilson, et al., 1994b, Shinn et al., 1998).

(248) Particle energy distributions measured on STS-48 are shown in Fig. 5.8. Charged particle telescopes are used which provide measurements of the energy spectra of light particles from about 15 MeV to 400 MeV for protons and other $Z=1$ and $Z=2$ ions from 5 MeV to 70 MeV/u (see above). These measurements are strictly secondary radiation due to the Earth's geomagnetic cut-offs, which exclude particles below a few hundred MeV/u from entering the spacecraft orbit. Excellent agreement with the HZETRN code for protons is found. For deuteron spectra the agreement is satisfactory only when knockout deuterons from proton and neutron induced reactions are included. For ^3He and ^4He the agreement is less satisfactory, and may point to a deficiency in the evaporation cross sections of the FLUKA model used by HZETRN.

(249) Detailed simulations of the radiation environment in the ISS from trapped proton radiation have been performed by Wilson et al. (2007) using the HZETRN code and Ersmark et al. (2007) using the Geant4 code. The anisotropy of the radiation field due to variation in shielding has also been studied.



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Fig. 5.8. Comparison of calculated (HZETRN code) and measured energy distributions of secondary protons (a) and deuterons (b) from GCR during STS-48 (Badhwar et al., 1995). Calculated proton and deuteron radiance (A) including knockout particles from proton and neutron induced reactions, (B) without those contributions.

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5.5 Radiation Shielding

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(250) Shielding models are applied for the spacecraft and spacesuits and allow evaluation of the interior environment to which the astronauts are exposed. The models describe the distribution of materials in the walls of and within the spacecraft and appropriate computational procedures are used to evaluate the interior field of transmitted particles. The internal environment, especially which based on protons from the SAA, shows a non-homogenous distribution with large spatial gradients and variations with time over both short-term and long-term temporal scales.

(251) With the exception of the absolute intensity of the trapped environment and possible solar particle events, the interior radiation environment can be well described using computational models. High-speed computational procedures allow rapid mapping of the interiors of the spacecraft.

(252) The interior environment of the spacecraft is also monitored by various instruments, which can be used to adjust the trapped-particle intensity, reduce the uncertainty in the model estimates, evaluate transmission factors, and evaluate calculated dosimetric quantities.

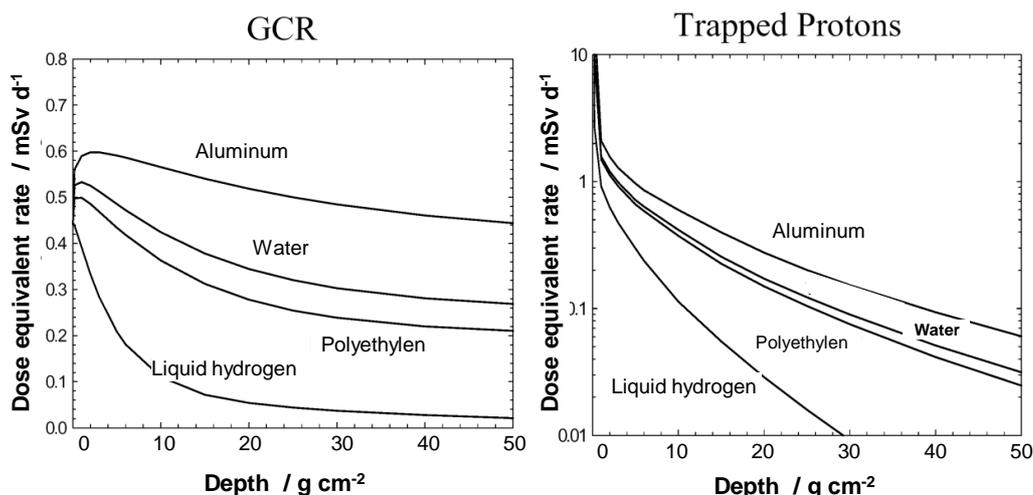
(253) Materials with light constituent atoms, such as hydrogen are most efficient per unit mass of material at slowing down ions, attenuating heavy ion projectiles through projectile fragmentation, and minimizing the build-up of neutrons and other target fragments produced directly from the shielding by nuclear interaction. Energy loss through ionisation is proportional to the number of electrons per atom (Z/A) where Z is the charge number and A the mass number, and the energy loss per unit mass is proportional to $(Z/\rho A)$ where ρ is the density of the material. For the GCR, materials such as aluminum (the most common spacecraft material) have relatively flat depth-dose equivalent responses due to the build-up of light particles in balance with the attenuation of heavy ions (Wilson *et al.*, 1995a). Materials, such as concrete or lead, have a response to the GCR that is predicted to increase the dose with shielding depth because of the large production of neutrons and target

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3293 fragments. Spacecraft materials are often deficient in hydrogen and therefore neutron spectra
 3294 will change appreciably in the first few cm of tissue as low-energy neutrons (< 5 MeV) are
 3295 produced more frequently compared to aluminum or other common spacecraft materials.

3296 (254) For a given area density and a given incident charged particle, ionisation energy loss
 3297 increases with the charge-to-mass ratio of the target nucleus (Z_T/A_T), while the fragmentation
 3298 cross section per unit mass is proportional to $A_T^{-1/3}$. Hence, hydrogen is the most efficient
 3299 material for shielding against heavy ions, and materials abundant in loosely bonded hydrogen
 3300 atoms are excellent candidates for efficient radiation shielding.

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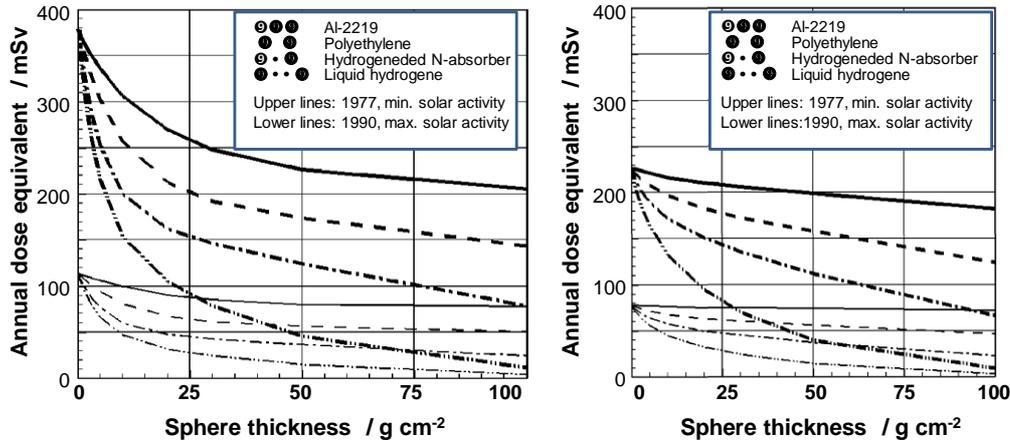


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 3303 Fig. 5.9. Calculated dose equivalent rate in tissue versus shielding thickness for different shielding
 3304 materials in the ISS orbit for GCR and trapped protons and solar minimum conditions (Cucinotta et
 3305 al., 2000b)

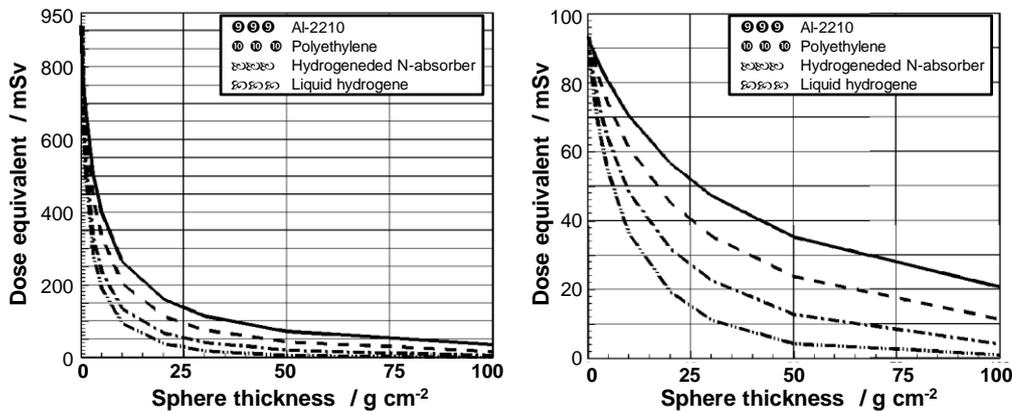
3306
 3307 (255) Having just one shielding material is an ideal case, which will in practice be very
 3308 hard to realize. The final shielding effectiveness will therefore also depend on the geometry
 3309 and the abundance of the various other materials used in the shielding. Ultimately, detailed
 3310 simulations will always be mandatory in evaluating and designing a realistic spacecraft or
 3311 habitat. Simulations suggest that shielding is effective against trapped protons in LEO, but its
 3312 efficiency is poor against GCR penetration. This is demonstrated clearly in Fig. 5.9.

3313 (256) In case of thicker shields, the neutrons, which as primary components of space
 3314 radiation are negligible, can become a noticeable source of radiation exposure. This occurs
 3315 not only in heavier shielded spacecraft but also on planetary or lunar surfaces which lack an
 3316 atmosphere thick enough to attenuate the primary radiation source to a reasonably low level.
 3317 On the surface of Mars and even more so of the Moon, these secondary “albedo” neutrons
 3318 emerging from the ground contribute significantly to the overall exposure, in particular, since
 3319 neutrons are high-LET radiation with a high radiobiological effectiveness.

3320 (257) There is an immense body of work already done in developing shielding strategies
 3321 for human space exploration missions. This activity resulted in numerous workshops and
 3322 publications in this field (Wilson et al., 1995a; Durante and Cucinotta, 2011). As mentioned
 3323 above, all calculations and measurements show that hydrogenous materials are the best
 3324 candidates per unit mass basis. The next set of figures (Fig. 5.10) taken from Wilson et al.
 3325 (2001) shows this clearly for cosmic ray spectra on Moon and Mars and a worst-case SPE.



(a) GCR at the Moon (left) and Mars (right)



(b) Worst Case SPE at the Moon (left) and Mars (right)

Fig.5.10. Set of calculations showing dose equivalents for different shielding materials versus shielding thickness on the surface of Moon (left) and Mars (right) for GCR (a) and a worst-case SPE (4x the fluence of the SPE on Sept. 1989) (b) (Wilson et al., 2001)

(258) Much of the protection inside a spacecraft is provided by structural elements and the equipment present in it. For the structure of the spacecraft, a compromise between shielding efficiency and mechanical stability needs to be chosen, with multifunctional materials needed to optimize the process. For a shelter inside the spacecraft there are not such restrictions – if the material is acceptable for space application – and the efficiency per mass is the only important endpoint. Some recent studies were performed for different shielding materials as shown in Fig 5.11 for vertical incidence (Zeitlin et al., 2006). The effect of the incidence angle of the particles in producing secondary neutrons is complex and exhibits a strong interaction with the type of the shielding material. Such studies of course need to be extended.

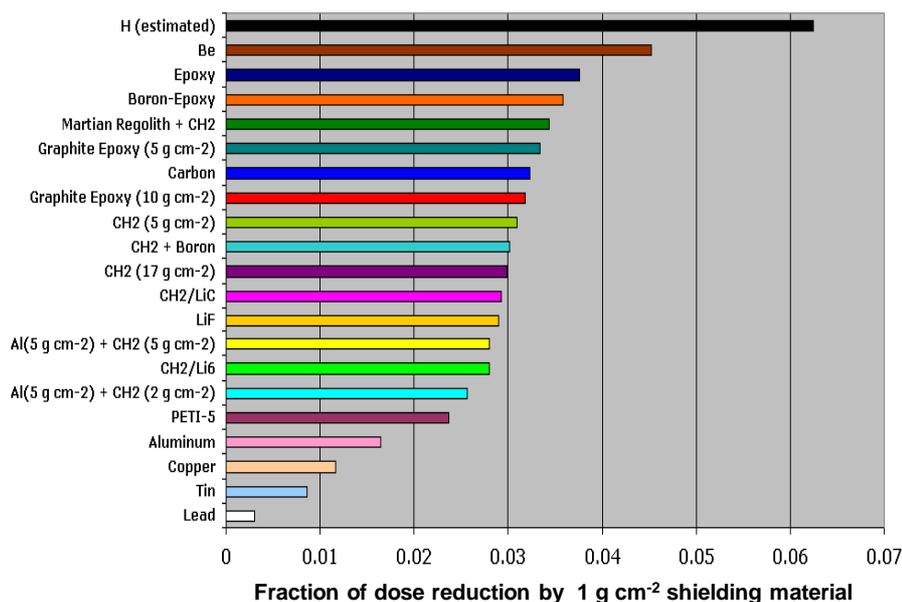


Fig 5.11. Fraction of dose reduction in a body by a 1 g cm⁻² thick shielding for different shielding materials and incident Fe particles of 1067 MeV/u. (Zeitlin et al., 2006)

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(259) Since the earliest attempts at in situ space radiation dosimetry, it has been known that - even for the most simple transport problems - satisfactory agreement between measurements and calculations would not be attainable unless the detailed thickness distribution around the point of interest were known, especially if areas of thinner shield were present. Therefore estimates of radiation exposures for ‘homogeneous’ ‘isotropic’ shields of an ‘average’ thickness can only serve for qualitative comparison of different configurations. For accurate quantitative assessments of radiation exposures, the knowledge of the distribution of the surrounding shield matter as a function of representative shield thickness is essential.

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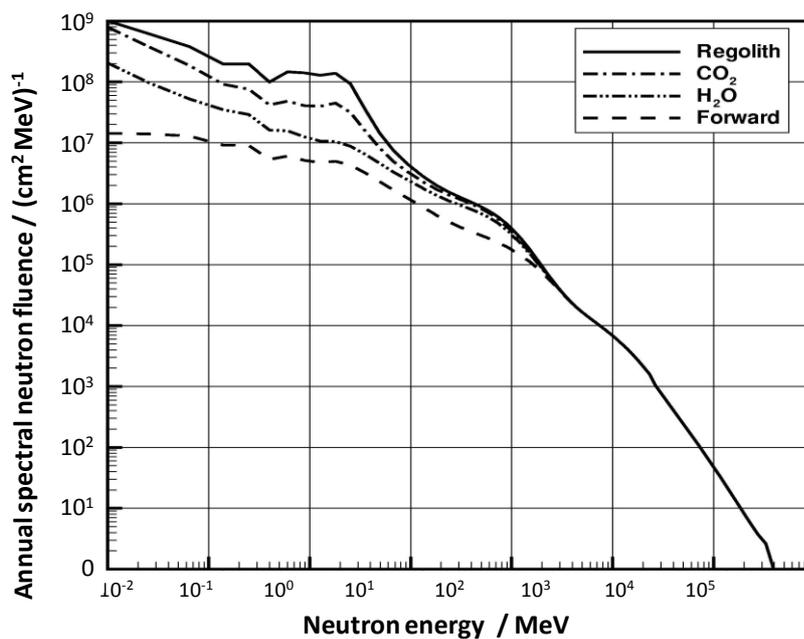
(260) Shielding to galactic cosmic radiation is generally limited. Nevertheless, with the selection of optimized shield material and an optimal inclusion of consumables in the shield design, a significant dose reduction of about 30 % can be achieved at solar minimum and to a lesser extent at solar maximum. For SPE’s the reduction is highly dependent on the initial proton spectrum; however shielding can reduce effective doses by factors of 2 to more than 10. As a first step definition, procurement and characterisation of candidate flexible materials – to be used in future manned missions in LEO and beyond, for inhabited structures has to be done. Computer codes are the tool to make the characterisation of such materials. The next step is the improvement and validation of the models and tools for shielding analysis, by comparison with measurements from accelerator shielding studies and with flight measurements, correlation and tuning of models, with the objective of reducing the overall uncertainty.

5.6 Lunar and Mars Surface

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(261) There are two effects strongly influencing the radiation environment on planetary or lunar surfaces compared to the GCR in free space. Because of the shielding of the ground, the cosmic radiation is incident on the surface only in 2π-geometry. In addition, from the surface of Mars or the Earth’s Moon, albedo radiation occurs. Photons, neutrons or secondary

3378 charged particles produced by neutrons will occur and will be dependent on the soil and
 3379 atmospheric (on Mars) atomic composition including the presence of CO₂ or water frost and
 3380 higher neutron fluence rate from lunar or Mars regolith (Cloudsley et al., 2000). Neutrons
 3381 may be divided into a forward component produced by GCR interactions with the atmosphere
 3382 and albedo components on planetary surfaces such as Mars. Albedo neutrons may be
 3383 produced as deep as 1 meter into the soil and their flux will be influenced by the soil
 3384 composition and seasonal variations in temperature as illustrated in Figure 5.12. Large dust
 3385 storms on Mars could lead to additional scattering of neutrons and charged particles (Wilson
 3386 et al., 1995a).



3387 Fig. 5.12 Energy distribution of neutron fluence on the Mars surface calculated with HZETRN
 3388 showing the annual contributions from the forward component produced by GCR in the Mars
 3389 atmosphere and the albedo component that are dependent on the composition of the Mars
 3390 surface (Wilson et al., 2004).
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3394 6. RADIATION FIELDS AND DOSES IN THE HUMAN BODY

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3397 (262) The radiation fields inside astronauts differ from those outside because of atomic
 3398 and nuclear interactions in the human body. The analysis of particle transport inside the body
 3399 is indispensable for the estimation of doses in astronauts and risks of both stochastic and
 3400 deterministic effects due to cosmic radiation exposures. Various simulation codes (see 5.3)
 3401 and computational phantoms, which represent the anatomy of human body or parts thereof,
 3402 have been employed for this analysis. In radiological protection, the mean absorbed dose in
 3403 an organ or tissue, D_T , is the basic quantity for a specification of doses in humans (see
 3404 Chapter 3). In addition, appropriate weighting factors need to be applied to D_T for the
 3405 assessment of risk of stochastic or deterministic detriments due to radiation exposure.

3406 (263) Generally, two different procedures may be applied for the assessment of doses in

3407 the human body, either by calculations or by measurements combined with calculations.
3408 Radiation field parameters, e. g. particle fluence, particle spectra and LET-distributions,
3409 outside or within a spacecraft may be determined either by measurements or calculations and
3410 then doses in organs and tissues of the human body may be calculated using particle transport
3411 codes. There are two possibilities in performing this task. One may either assess the radiation
3412 field parameters (e.g. energy distribution of fluence, $D(L)$ -distributions etc.) (Section 5.3)
3413 near to an astronaut and then apply fluence-to-dose conversion coefficients for all types of
3414 particles involved for the assessment of organ doses (see Sect. 6.2) or one can generally
3415 calculate organ doses in a body using the radiation field data outside of the spacecraft and a
3416 code which combines radiation transport in the spacecraft and in the human body (see Sect.
3417 6.3).

3418 (264) Alternatively, absorbed dose or dose equivalent may be measured near to the body
3419 of the person of interest and these values may be directly correlated to doses in the human
3420 body. This is the usual procedure performed in individual dosimetry on the Earth, where the
3421 reading of an individual dosimeter for strongly-penetrating radiation is taken as a value of
3422 effective dose sufficiently precise for the purpose of usual radiological protection. In space,
3423 however, this method is a difficult task because of the very complex radiation field, which
3424 also shows variations with time and position within a spacecraft. As shown in Chapter 4, no
3425 single device will probably be able to fulfil this task, and a set of different detectors may be
3426 necessary for the assessment of dose equivalent in an organ or tissue or effective dose
3427 equivalent. The position and orientation of a person within the spacecraft can introduce
3428 variations in organ doses due to the anisotropic spacecraft shielding distributions, which can
3429 be important for solar protons and trapped radiation (Wilson et al., 1995c). In any case,
3430 particle transport calculations need to be used for testing if a system is appropriate for the
3431 foreseen task.

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3433 **6.1 Phantoms**

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3435 (265) The first computational phantom was developed at the Oak Ridge National
3436 Laboratory (Snyder et al., 1969) for the Medical Internal Radiation Dose (MIRD) Committee
3437 of the Society of Nuclear Medicine. The MIRD phantom designed as a hermaphrodite
3438 including organs and tissues of both sexes has been widely used in varieties of the radiation
3439 protection researches, in combination with various Monte Carlo codes. On the other hand, the
3440 Computerized Anatomical Man (CAM) (Billings and Yucker, 1973) and Computerized
3441 Anatomical Female (CAF) (Yucker and Huston, 1990) phantoms were developed in 1973 and
3442 1990, respectively. They have a long history in space radiation research at NASA, in
3443 combination with the transport code HZETRN. These phantoms are based upon mathematical
3444 expressions representing planes, and cylindrical, conical, elliptical and spherical surfaces that
3445 describe the shape and position of idealized body organs.

3446 (266) As an extension and improvement to these earlier models, various groups have
3447 developed a new type of anatomical phantom during the last two decades, called
3448 “tomographic” or “voxel” phantom. Voxel phantoms are anatomical models based on
3449 computed tomography, magnetic resonance, or other images obtained from high-resolution
3450 scans of a single individual and, thus, offer a more realistic replication of human anatomy.
3451 They consist of a large number of volume elements (voxels) and are the most detailed
3452 representation of human anatomy at the present time. However, being derived from a specific
3453 individual, these models do not represent the average Caucasian man or woman as defined by
3454 Publication 23 (ICRP, 1975) and Publication 89 (ICRP, 2002). To avoid this inconsistency,

3455 the Commission introduced reference voxel phantoms representing the adult Reference Male
 3456 and Reference Female defined in Publication 110 (ICRP, 2009), which were constructed
 3457 based on medical image data of real persons, but their anatomical parameters were modified
 3458 to be consistent with those given in Publication 89. These phantoms are used by the
 3459 Commission in establishing radiation protection guidance and reference data, e.g. conversion
 3460 coefficients for dosimetric quantities. NASA has used the voxel approach based on the
 3461 MAX/FAX voxel model in a ray tracing approach appropriate for the HZETRN code (Slaba
 3462 et al. 2010, Kim et al., 2010b)

3463 (267) One limitation of the phantoms is that their resolutions are not high enough to
 3464 reproduce the thin structure of a tissue or organ located at or near to the surface of human
 3465 body, e.g. skin and lens of the eye. This causes both overestimation and underestimation of
 3466 doses in such tissue and organ for the irradiation by low-energy particles such as trapped
 3467 protons and electrons. Thus, special procedures are required to precisely calculate the dose in
 3468 such tissues or organs, using the phantoms. A more detailed description on this issue is given
 3469 in Publication 110 (ICRP, 2009).

3470 (268) For realization of anthropomorphic phantoms, several materials such as water and
 3471 tissue-equivalent plastics are used on ground. On the other hand, only solid plastic phantoms
 3472 can be launched to space for practical reasons. The RANDO[®] phantoms of head and upper
 3473 torso had been mounted on the Space Shuttle (Konradi et al., 1992, Yasuda et al., 2000,
 3474 Badhwar et al., 2002) and the ISS (Cucinotta et al., 2008). The other RANDO[®] phantom had
 3475 been exposed outside the ISS as a part of the MATROSHKA project (Reitz et al., 2009). A
 3476 spherical phantom composed of tissue-equivalent plastic has also been mounted on the ISS as
 3477 part of the MATROSHKA-R project (Shurshakov et al., 2004). A number of passive and
 3478 active detectors were inserted in or attached on the phantoms. The data obtained from the
 3479 detectors are useful in validating the accuracy of particle transport simulations performed
 3480 using the computational phantoms.

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3482 6.2 Dose conversion coefficients

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3484 Mean absorbed dose in organs and tissues

3485 (269) For a given organ T and radiation type R, the mean absorbed dose in the organ, $D_{T,R}$,
 3486 can be obtained folding the energy distribution of the fluence of particles incident on the
 3487 human body with the fluence-to-absorbed dose conversion coefficients for that organ,
 3488 $d_{T,R}(E)$. It is:

3489
$$D_{T,R} = \int_0^{\infty} d_{T,R}(E) \frac{d\Phi_R}{dE} dE \quad , \quad (6.1)$$

3490 where Φ_R is the fluence of particles of type R incident on the body. Because conversion
 3491 coefficients, $d_{T,R}$, are mainly available for a homogeneous exposure of the human body, only,
 3492 the application of this equation assumes always a uniform exposure of the body. In cases
 3493 where this assumption is not approximately satisfied, further considerations are needed for
 3494 the application of this approach.

3495 (270) For assessing organ doses it is necessary to have conversion coefficients for all
 3496 organs and tissues of the human body, for all particles and energies of interest and for the real
 3497 geometry of irradiation.

3498 (271) For the calculation of conversion coefficients, an exposure of a mathematical
 3499 anthropomorphic phantom is simulated for the incidence of monoenergetic particles
 3500 according to simple geometries, mostly homogeneous frontal incidence (AP), incidence from

3501 the right or left side (RLAT, LLAT), from the back (PA) or rotational (ROT) and isotropic
 3502 (ISO) exposure. The isotropic irradiation is usually assumed to appropriately describe the
 3503 exposure of astronauts to cosmic radiation, mainly due to the isotropic fluence rate of the
 3504 GCR and the movement of the astronauts in the spacecraft. Nevertheless, this assumption
 3505 may not always represent the situation well (Wilson et al., 1995c).

3506 (272) The absorbed dose in the various organs and tissues of the human body are
 3507 estimated from the energies deposited in the regions assigned to each organ divided by their
 3508 masses. In the case of the voxel phantoms the mean absorbed doses in the organs are
 3509 estimated from the energies deposited in the voxels assigned to each organ divided by the
 3510 organ mass. This method has been applied in most organs, including skin, for the calculations
 3511 of the conversion coefficients adopted by the Commission. Exceptions are the red bone
 3512 marrow and bone surface (endosteum), which are tissues not explicitly defined in the
 3513 reference phantoms. According to Schlattl et al. (Schlattl et al., 2007), the mean absorbed dose
 3514 in red bone marrow, D_{RBM} , and in endosteum, $D_{\text{Endosteum}}$, are determined respectively by

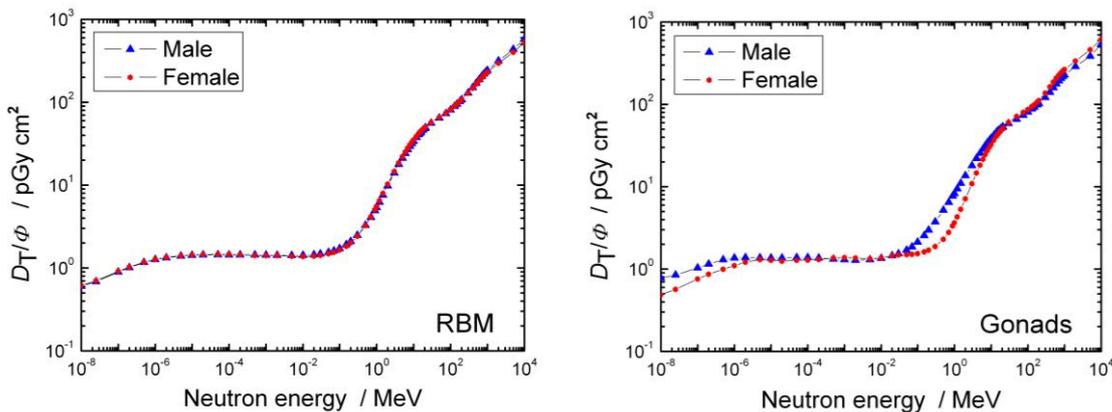
3515
$$D_{\text{RBM}} = \sum_i \frac{m_{\text{RBM},i}}{m_{\text{RBM}}} D_{\text{spongiosa},i} \quad (6.2)$$

3516 and

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$$D_{\text{Endosteum}} = \sum_i \frac{m_{\text{Endosteum},i}}{m_{\text{Endosteum}}} D_{\text{spongiosa},i} \quad (6.3)$$

3518 where $m_{\text{RBM},i}$ is the mass of RBM in i-th spongiosa region, m_{RBM} the total mass of RBM,
 3519 $D_{\text{spongiosa},i}$ the dose of i-th spongiosa region, $m_{\text{Endosteum},i}$ the mass of endosteum in i-th
 3520 spongiosa region and $m_{\text{Endosteum}}$ the total mass of endosteum.

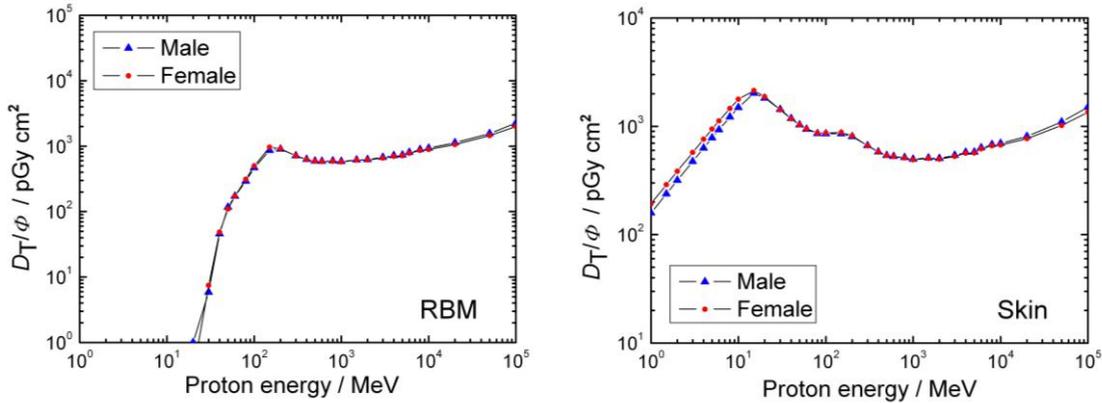
3521 (273) As an example, conversion coefficients for red bone marrow and gonads,
 3522 respectively, published by the Commission for isotropic irradiation of the reference adult
 3523 male and female phantoms are shown in Fig. 6.1 as a function of neutron energy (ICRP,
 3524 2012).



3525 Fig. 6.1. Fluence to mean absorbed dose conversion coefficients for red bone marrow (left) and
 3526 gonads (right) as a function of neutron energy for isotropic irradiation of the adult male and female
 3527 reference phantoms (ICRP, 2012).
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3530 (274) Figure 6.2 shows similar conversion coefficients for red bone marrow and skin,
 3531 respectively, as a function of energy for protons (ICRP, 2012). It should be noted that the
 3532 gender dependence of the organ doses are significant only for few organs and varies with the
 3533 type and energy of the radiation involved.
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Fig. 6.2. Fluence to mean absorbed dose conversion coefficients for red bone marrow (left) and skin (right) as a function of proton energy for isotropic irradiation of the adult male and female reference phantoms (ICRP, 2012).

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(275) The fluence to mean absorbed dose conversion coefficients for organs and tissues of the body have been systematically calculated for heavy ions with atomic numbers up to 28 and energies from 1 MeV/u to 100 GeV/u using the PHITS code coupled to the reference voxel phantoms (Sato et al., 2010), following the instruction given in Publication 103 (ICRP, 2007). A full set of data for male and female and isotropic exposure of the body is given in the Annex of this report.

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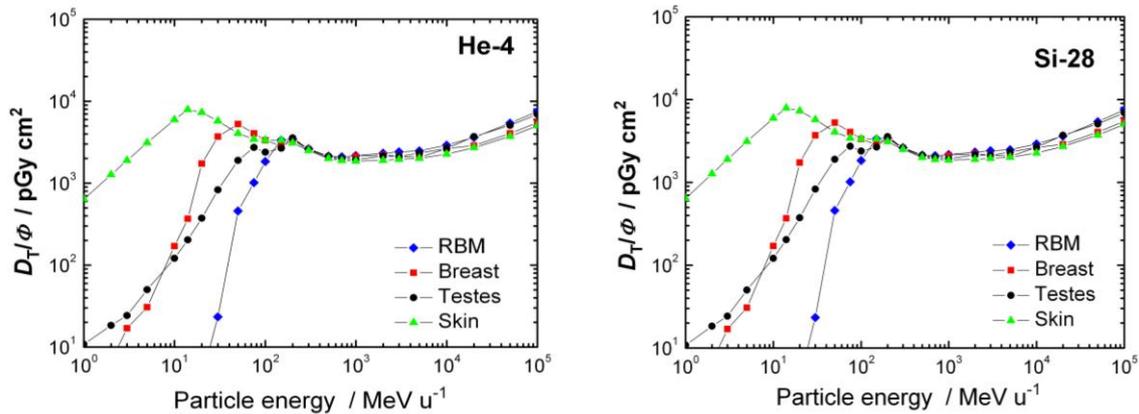
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(276) As an example, the data for red bone marrow (RBM), breast, stomach and skin are plotted in Figures 6.3 for isotropic (ISO) irradiation of the reference adult male phantom with ^4He and ^{28}Si ions, respectively.



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Fig. 6.3. Fluence to mean absorbed dose conversion coefficients for various tissues as a function of particle energy for isotropic irradiation of the reference adult male phantom by ^4He ions (left) and ^{28}Si (right).

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(277) It can be seen from figure 6.3 that the dose conversion coefficients for the organs are very different for the energies below about 200 MeV, while they are very similar for high-energies. At low energies, the incident particles have short ranges in tissue and will generally stop in an organ or tissue near to the surface (i.e. skin). At high energies, on the other hand,

3561 the incident particles have very long ranges and generally penetrate the human body without
 3562 forming the Bragg peak. Thus, the doses are more uniformly distributed inside the human
 3563 body in comparison to low-energy particle irradiations.

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3565 **Organ and effective dose equivalent**

3566 (278) While fluence to mean absorbed dose conversion coefficients for organs and tissues,
 3567 $d_{T,R}$, are the basis for dose assessment in the human body, in mixed radiation fields with
 3568 radiations of very different radiation quality fluence to dose equivalent conversion
 3569 coefficients, $h_{T,Q,R}$, for organs and tissues are often more appropriate for radiation protection
 3570 applications and risk assessments in space environment. Most space organizations (e.g.
 3571 NASA, ESA, etc.) have adopted these quantities and corresponding conversion coefficients
 3572 since the late 1990s (NCRP, 2000).

3573 (279) The calculation of the conversion coefficients for mean dose equivalents in organs
 3574 and tissues and effective dose equivalent, are more complex than those for the absorbed
 3575 doses, because the doses at the point of interest must be weighted by the quality factor as a
 3576 function of LET. In this case, the charge and energy of ionizing particles depositing the
 3577 energy at that point have to be determined. If kerma factors are used for determining the
 3578 mean absorbed dose in tissues and organs (e.g. for neutrons below about 20 MeV) this does
 3579 not allow the evaluation of a Q -value unless one includes further corrections.

3580 (280) The dose equivalent in an organ or tissue T, $H_{T,Q}$, is calculated by:

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$$H_{T,Q} = \sum_R \int_E h_{T,Q,R}(E) \frac{d\Phi_E}{dE} dE = \sum_R \int_E Q_{T,R}(E) d_{T,R}(E) \frac{d\Phi_E}{dE} dE \quad (6.4)$$

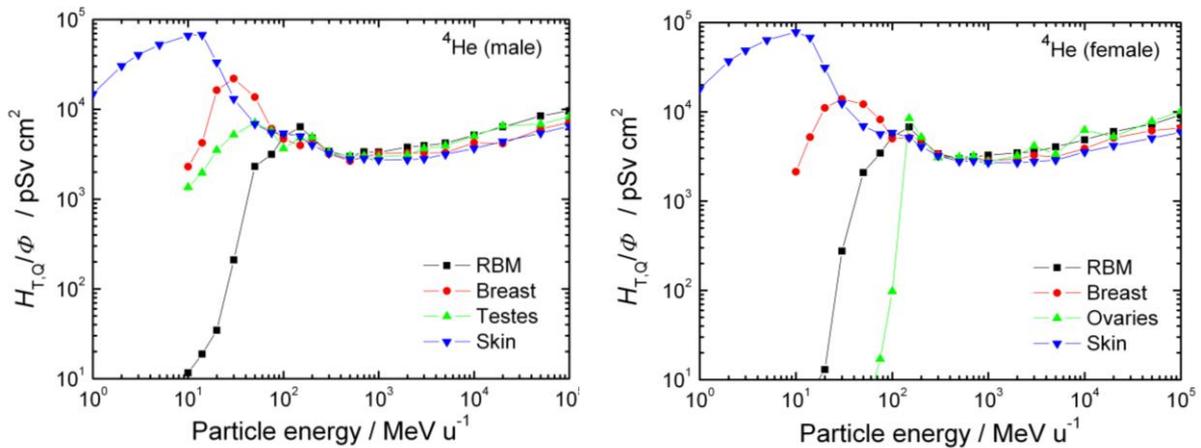
3582 with the mean quality factor $Q_{T,R}$ based on $Q(L)$ as given in eq. (3.10).

3583 (281) The mean quality factors, $Q_{T,R}$, and the fluence to organ dose equivalent conversion
 3584 coefficients, $h_{T,Q,R}$ for organs and tissues have been systematically calculated for heavy ions
 3585 with atomic numbers up to 28 and energies from 1 MeV/u to 100 GeV/u using the PHITS
 3586 code coupled to the reference voxel phantoms (Sato et al., 2010). Data of mean quality
 3587 factors for organs and tissues and isotropic exposure of the body are also given in the Annex
 3588 to this report.

3589 (282) The organ dose equivalent conversion coefficients obtained by PHITS for red bone
 3590 marrow, breast, gonads (testes and ovaries, respectively) and skin of the adult male and
 3591 female reference phantoms are depicted in Fig. 6.4 and 6.5 for ISO irradiation by ^4He and
 3592 ^{56}Fe , respectively.

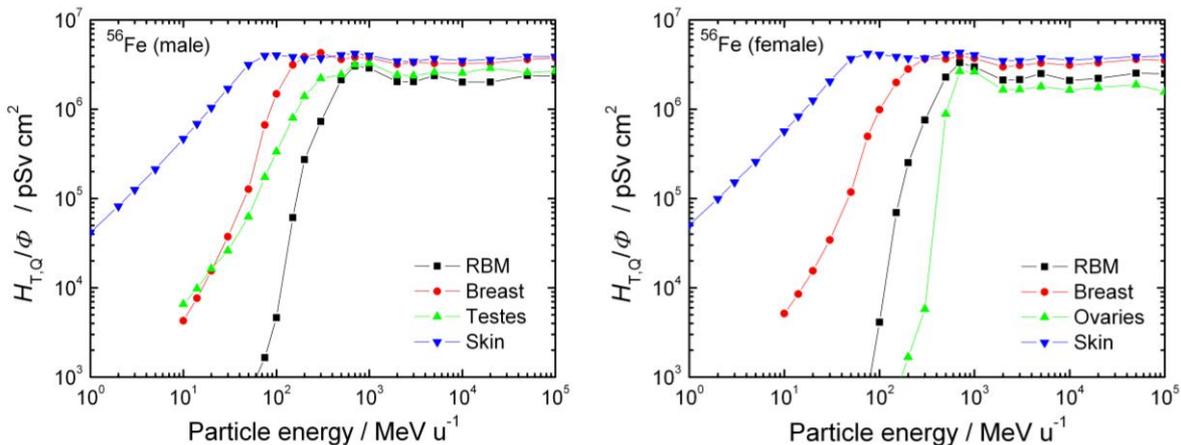
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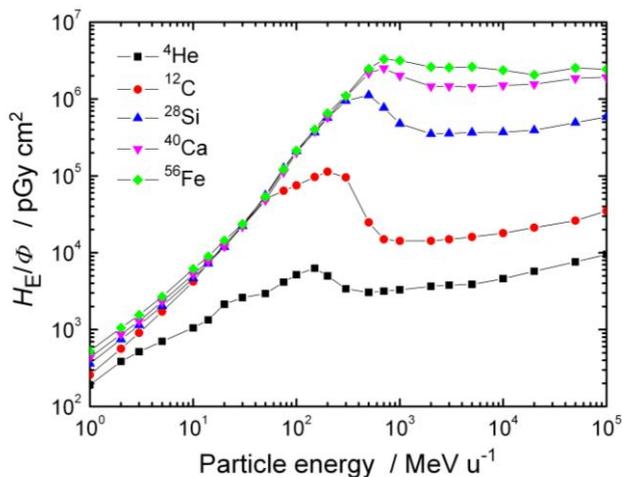
3598 Fig. 6.4. Fluence to organ dose equivalent conversion coefficients for various tissues as a function
 3599 of the particle energy for ⁴He ions and isotropic irradiation of the adult male (left) and female (right)
 3600 reference phantom.



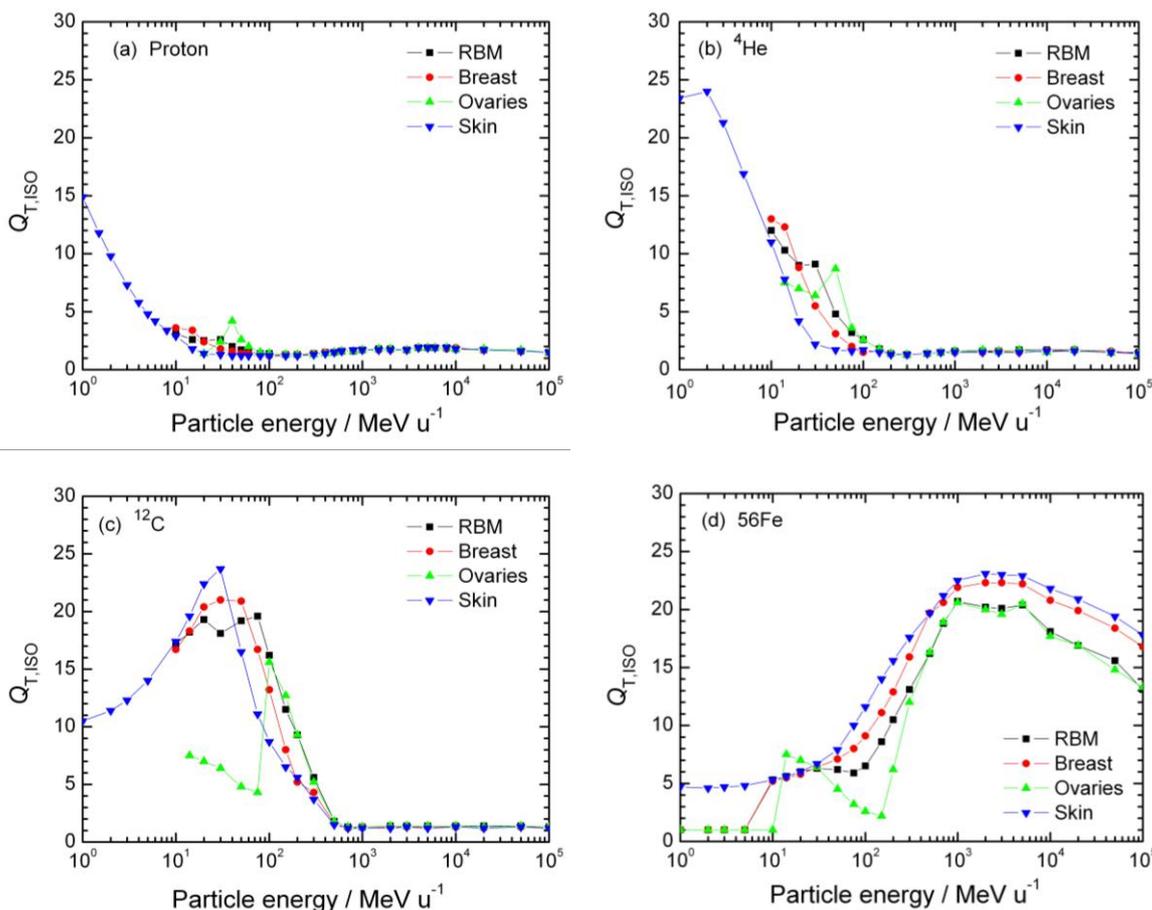
3601

3602 Fig. 6.5. Fluence to organ dose equivalent conversion coefficients for various tissues as a function
 3603 of the particle energy for ⁵⁶Fe ions and isotropic irradiation of the adult male (left) and female (right)
 3604 reference phantom.
 3605

3606 (283) As far as fluence-to-effective dose equivalent conversion coefficients are concerned,
 3607 the data for neutrons are very similar to those for effective dose. Larger differences between
 3608 conversion coefficients for effective dose and effective dose equivalent exist for low energy
 3609 protons, where the value of the mean quality factor is much higher than the value of 2 for the
 3610 radiation weighting factor. In high-energy proton fields, however, this difference is not very
 3611 important because of the small contribution of low-energy protons to effective dose and
 3612 effective dose equivalent. Fig. 6.6 presents some data for several heavy ions.



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 3614 Fig. 6.6. Fluence to effective dose equivalent conversion coefficients for several particles as a
 3615 function of the particle energy for isotropic irradiation of adult male reference phantom.
 3616

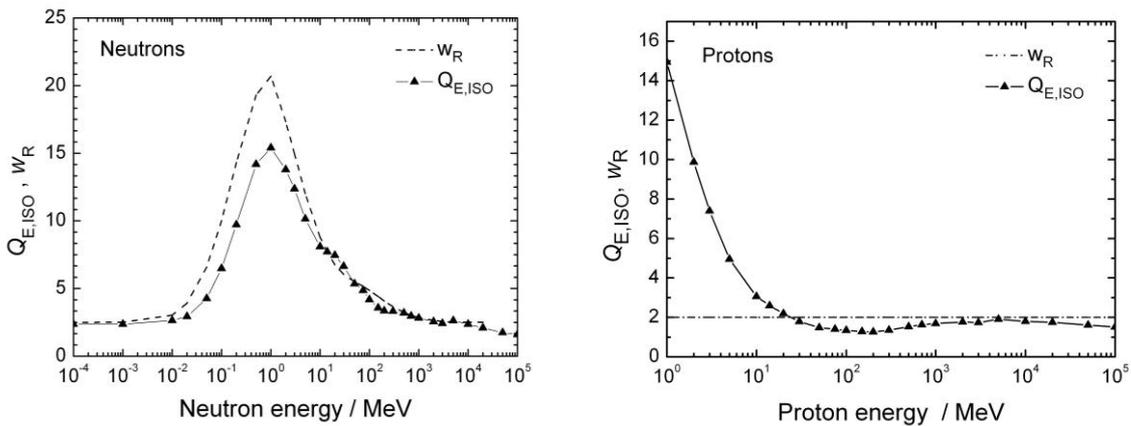


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 3618
 3619 Fig. 6.7. Mean quality factors, $Q_{T,ISO}$, based on $Q(L)$ as defined in Publication 103, of some tissues
 3620 as a function of particle energy for isotropic exposure of the adult female reference phantom by (a) p,
 3621 (b) ^4He , (c) ^{12}C and (d) ^{56}Fe .
 3622

3623 (284) The ratio of the mean dose equivalent in an organ to the mean absorbed dose in that

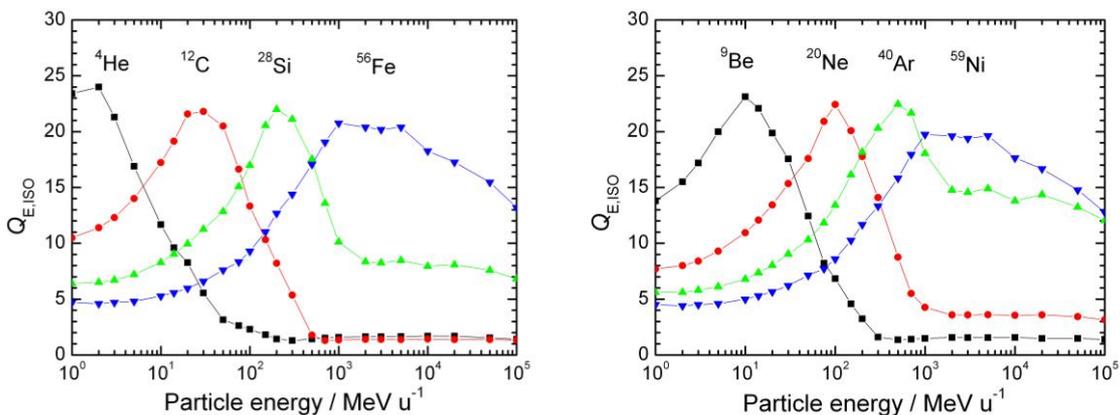
3624 organ gives the mean quality factor for that organ. As an example, Figures 6.7 a-d shows the
 3625 mean quality factors for red bone marrow, breast, stomach and skin for exposure of the adult
 3626 female reference phantom by protons, ^4He -, ^{12}C - and ^{56}Fe -ions.

3627 (285) As can be seen in Fig. 6.7, for isotropic exposure to a specific ion type the
 3628 differences in Q_T of the different organs and tissues are relatively small. Therefore, a human-
 3629 body averaged quality factor (performed by weighting the organs and tissues (see eq. (3.12))
 3630 using the tissue weighting factors given by the Commission (ICRP, 2007)) represents the
 3631 radiation quality well in isotropic exposure situations.
 3632



3633 (a) (b)
 3634 Fig. 6.8. Phantom averaged quality factor, $Q_{E,ISO}$, as a function of neutron energy (a) and proton
 3635 energy (b) for isotropic (ISO) exposure of the adult male reference phantom. The w_R -functions are
 3636 additionally shown (dotted line).
 3637
 3638

3639 (286) While for AP radiation incidence body averaged quality factors for neutrons and
 3640 protons are shown in Figs. 3.3 and 3.4, for data for isotropic exposure are given in the Annex
 3641 and presented in Fig. 6.8a,b. In Fig. 6.9a, b similar data are shown for various heavy ions.

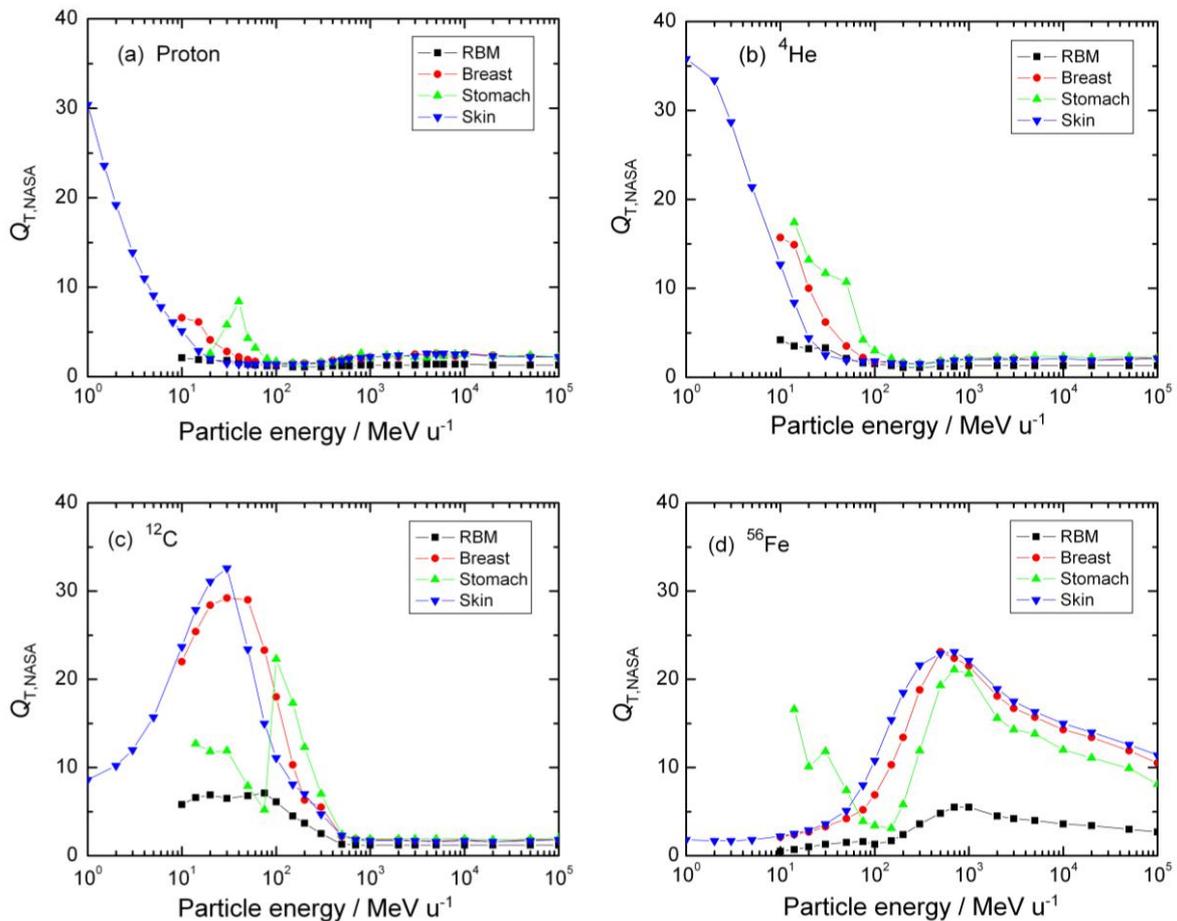


3642 Fig. 6.9. Phantom averaged quality factor, $Q_{E,ISO}$, as function of particle energy for various particles
 3643 and isotropic exposure of the adult male reference phantom.
 3644
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3646 (287) Mean quality factors for organs and tissues, $Q_{T,NASA}$, based on the Q -function as
 3647 proposed by NASA for solid cancer (see Fig. 3.14), were also calculated using the PHITS
 3648 code coupled to the reference voxel phantoms (Sato et al., 2012). As an example, figures 6.10

3649 a-d show $Q_{T,NASA}$ of the red bone marrow, breast, stomach and skin for isotropic exposure by
 3650 protons, ^4He , ^{12}C and ^{56}Fe ions. The values of red bone marrow were calculated using the
 3651 NASA quality factor for leukaemia which is 1/4 of that for solid cancer.

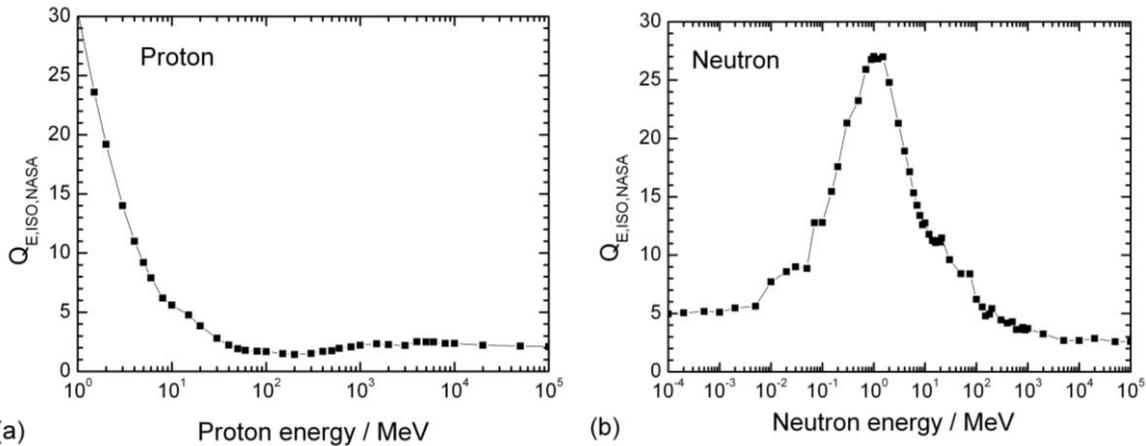
3652 (288) For proton and light ions, the values of $Q_{T,NASA}$ are generally larger than the
 3653 corresponding data based on the $Q(L)$ function given in eq. (3.9) ICRP (Fig. 6.7), except for
 3654 red bone marrow. For heavier ions, however, the values of $Q_{T,NASA}$ are generally smaller than
 3655 the corresponding values based on the ICRP function especially for very low and very high
 3656 particle energies. The smaller $Q_{T,NASA}$ observed at low energies is attributed to the strong
 3657 decrease of $Q_{T,NASA}$ with increasing L in the high-LET region, while for high particle energies
 3658 the consideration of the track structure in the NASA concept is important.



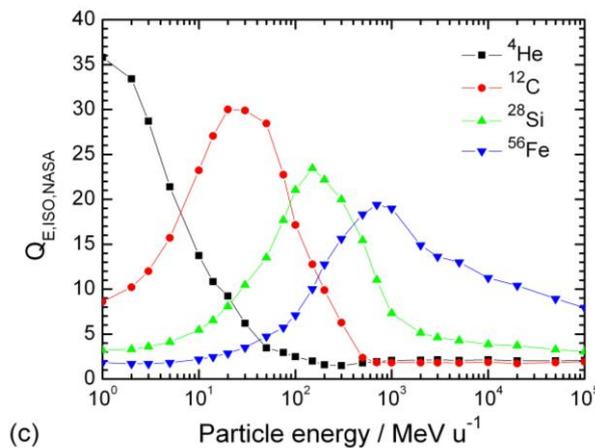
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 3661 Fig. 6.10. Mean quality factors, $Q_{T,NASA}$, based on the Q -function proposed by NASA, of some
 3662 organs as a function of particle energy for isotropic exposure of the adult female reference phantom
 3663 by (a) protons, (b) ^4He , (c) ^{12}C and (d) ^{56}Fe .
 3664

3665 (289) Figure 6.11 shows human body averaged quality factors based on $Q_{T,NASA}$ for
 3666 isotropic exposure of the adult female reference phantom by various particles. The tissue
 3667 weighting factor, w_T , as defined in Publication 103 (ICRP, 2007) were adopted in the
 3668 calculation.
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3672 Fig. 6.11 Phantom averaged quality factor, $Q_{E,ISO,NASA}$, as function of particle energy for protons
 3673 (a), neutrons (b) and various other ions (c) and isotropic exposure the adult male reference phantom.

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3675 6.3 Calculation of organ doses of astronauts within spacecraft

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3677 (290) Generally, organ doses in a body may be calculated using the radiation field
 3678 parameters outside the spacecraft and a code which combines radiation transport in the
 3679 spacecraft and in the human body. In principle, this procedure takes account of all
 3680 inhomogeneities of the radiation field inside the spacecraft due to the variations in the
 3681 spacecraft wall and the distribution of the material inside the spacecraft. The approach has
 3682 been realized by NASA using the very fast radiation transport code HZETRN together with
 3683 various human phantoms (see e.g. Wilson, et al., 1995c, Badhwar et al., 2002, Cucinotta et al.,
 3684 2008). The external radiation field parameters to be known include the energy and directional
 3685 distribution of the fluence of all types of radiation involved. Also the time dependence of
 3686 some contributions needs to be known. This approach can be applied to the field for EVA and
 3687 for astronauts in a habitat on the moon or planet.

3688 (291) For checking this method, phantom torsos comprised of realistic distributions of
 3689 human tissue equivalent materials have been flown on several space shuttle missions. Organ
 3690 dose equivalents have been estimated by using a combined TLD and PNTD (CR-39) detector
 3691 methodology (Badhwar et al., 2002; Yasuda et al., 2000). Table 6.1 shows a comparison of
 3692 data calculated using the HZETRN/QMSFRG model (Cucinotta et al., 2008) to those from

3693 measurements of Yasuda et al. (2000) on space shuttle mission STS-91, which flew in a 51.6
 3694 inclination orbit to the Mir station a similar orbit as flown by ISS. The corresponding data
 3695 calculated by PHITS coupled with the dose conversion coefficients for isotropic irradiation
 3696 are also given in the table (Sato et al., 2011). The comparison shows excellent agreement
 3697 between measured and calculated data. The NASA phantom torso experiment that was flown
 3698 on STS-91 (Badhwar et al., 2002), was re-flown on ISS Increment 2 in 2001. This experiment
 3699 included several small active silicon detectors located at critical organ positions in the torso
 3700 that provide time dependent dose data. The correlation of the time dependent data to the ISS
 3701 trajectory allows for separation of the individual contributions from trapped protons and GCR
 3702 to organ doses. Table 6.2 shows comparison of the HZETRN/QMSFRG results (without
 3703 scaling) to the measurements indicating good agreement. The results show that the ratio of
 3704 the GCR to trapped proton absorbed dose is about 2:1. Mean quality factors without tissue
 3705 shielding for GCR (~3.5) are more than twice as high as that for the trapped protons (~1.5).
 3706 These results support the assumption that organ dose equivalents for ISS missions and many
 3707 space shuttle missions are predominantly from GCR.

3708 Table 6.1. Comparison of measured organ dose equivalent for STS-91 mission using a combined
 3709 PNTD/TLD method and data calculated using the HZETRN/QMSFRG model with random
 3710 orientation in the spacecraft (Yasuda et al., 2000), as well as using PHITS coupled with the dose
 3711 conversion coefficients (DCC) for isotropic irradiation (Sato et al., 2011).

Tissue	Organ dose equivalent / mSv				
	Measured	HZETRN/ QMSFRG	Diff. %	PHITS/ DCC	Diff. %
Skin	4.5 ±0.05	4.7	4.4	5.3	18.5
Thyroid	4.0 ±0.21	4.0	0	4.2	4.9
Bone surface	5.2 ±0.22	4.0	-23.1	4.3	-17.8
Esophagus	3.4 ±0.49	3.7	8.8	3.6	5.4
Lung	4.4 ±0.76	3.8	-13.6	3.9	-12.3
Stomach	4.3 ±0.94	3.6	-16.3	3.5	-17.5
Liver	4.0 ±0.51	3.7	-7.5	3.6	-10.7
Bone marrow	3.4 ±0.40	3.9	14.7	3.7	9.5
Colon	3.6 ±0.42	3.9	8.3	3.7	1.9
Bladder	3.6 ±0.24	3.5	-2.8	3.5	-2.3
Gonad	4.7 ±0.71	3.9	-17.0	4.2	-10.3
Breast	4.5 ±0.11	4.5	0	5.2	16.2
Remainder	4.0 ±0.57	4.0	0	3.7	-6.5
Effective dose equivalent	4.1 ±0.22	3.9	4.9	3.9	4.9

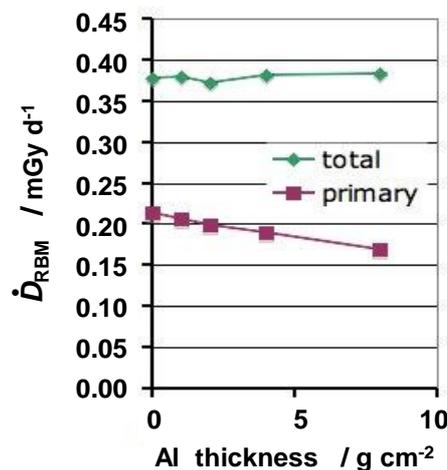
3712
 3713
 3714 Table 6.2. Comparison of the organ absorbed dose rate measured by the ISS Increment-2 Phantom
 3715 Torso experiment (July-August, 2001) with predictions obtained using the HZETRN/QMSFRG
 3716 model at a fixed position in the spacecraft (Badhwar *et al.*, 2002).
 3717

Organ	Absorbed dose rate trapped radiation mGy/d		Absorbed dose rate from GCR mGy/d		Total absorbed dose rate mGy/d		Difference (%)
	Exp.	Model	Exp.	Model	Exp.	Model	
Brain	0.051	0.066	0.076	0.077	0.127	0.143	13.3

Thyroid	0.062	0.072	0.074	0.077	0.136	0.148	9.4
Heart	0.054	0.061	0.075	0.076	0.129	0.137	6.7
Stomach	0.050	0.057	0.076	0.077	0.126	0.133	5.5
Colon	0.055	0.056	0.073	0.076	0.128	0.131	2.5

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(292) The doses received in interplanetary missions have been evaluated by the same approach by a number of authors (i.e. Badhwar et al., 1994; Cucinotta and Durante, 2006a, Hoff et al., 2004, Zapp et al., 2002, Ballarini et al., 2006, Trovati et al., 2006). The FLUKA code associated with a Golem voxel phantom (Zankl et al., 2001) was used for an estimate of the doses inside a capsule-like enclosure with variable Al-wall thickness (Ferrari, 2007). It was supposed that particles of GCR impinge uniformly and isotropically on the spacecraft. As an example, Figure 6.12 shows calculated results in terms of absorbed dose rate to red bone marrow (RBM) as a function of the thickness of the Al wall. The contribution of the primary particles to the total dose rates is also shown in Figure 6.12.



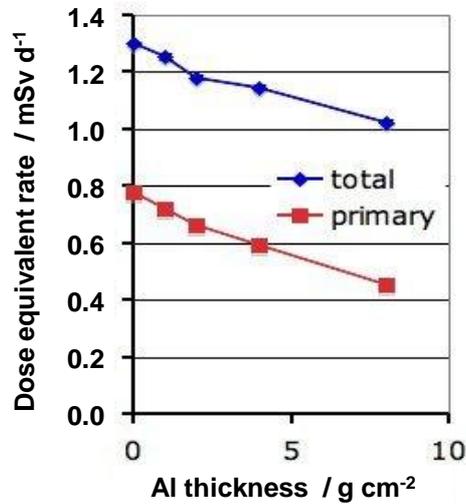
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Fig.6.12. Absorbed dose rate to red bone marrow (RBM) of a voxel phantom exposed to galactic cosmic radiation inside a capsule as a function of the Al wall thickness of the capsule (Ferrari, 2007).

(293) The calculated absorbed dose rate behind 1 g cm⁻² of Al resulted in 0.378 mGy/d, with 59% due to protons, 21% to alpha particles, 10% to ions of 2 < Z ≤ 8, 5% to ions of 8 < Z ≤ 14 and 5% to ions of Z > 14. The dose rate contributions of the various ion groups include those of primary ions and of all products generated in their interactions. The dose rate contribution of the uncollided particles was 0.206 mGy/d.

(294) Examples of direct evaluation of the organ dose equivalent rates from external radiation fluence rates of GCR without use of conversion coefficients are shown in Fig. 6.13. Similar to simulations already described above (see Fig. 6.12) the figure shows the dose equivalent rate to RBM as a function of the Al wall thickness of a capsule (Ferrari, 2007). It should be noted that for these data the tissue weighting factors recommended in the Publication 60 (ICRP, 1991) were applied.

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3747 Fig 6.13. Dose equivalent rate to the RBM of a voxel phantom inside a capsule exposed to galactic
3748 cosmic radiation as a function of the Al wall thickness of the capsule (data from Ferrari, 2007).
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(295) The calculated RBM dose equivalent rate behind 1 g cm⁻² of Al is 1.26 mSv/d, where 31% are due to protons, 11% to alpha particles, 13% to ions of 2 < Z ≤ 8, 16% to ions of 8 < Z ≤ 14, 29% to ions of Z > 14. The contributions of the various ion groups include the primary ions contributions and those of all products generated in their interactions. The contribution of the uncollided particles was 0.72 mSv/d.

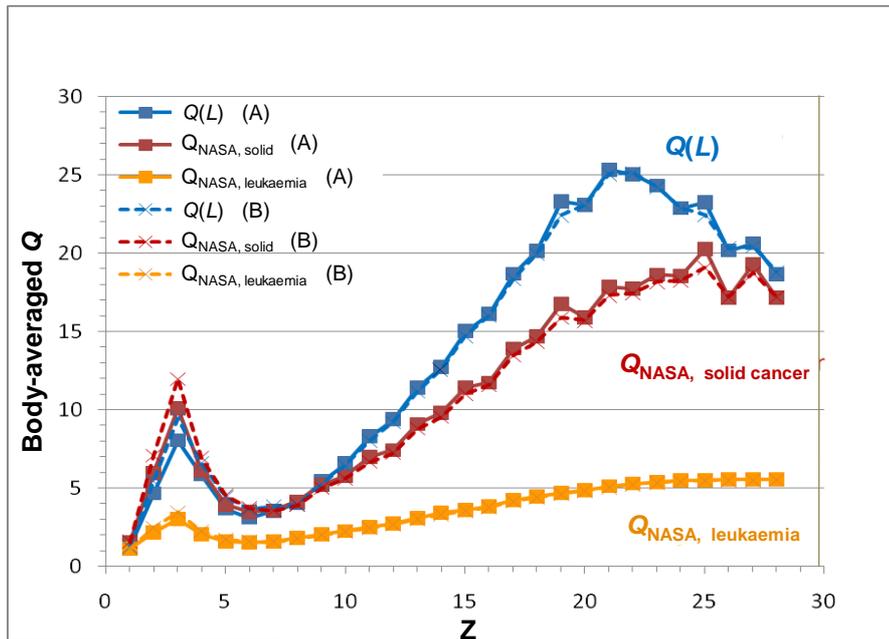
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(296) Several other authors used the method described above in order to determine the organ dose equivalent and effective dose equivalent received by the astronauts, especially for studying the shielding effect of the wall thickness of the capsule. Slaba et al. (Slaba et al., 2010) computed the protection quantities under both a galactic cosmic radiation and solar particle event environment. The spherical shell of aluminum was supposed isotropically irradiated and various computational models (CAM, CAF, MAX, FAX) have been used. Ballarini et al. (2006) and Trovati et al. (2006) calculated the GCR and SPE organ doses in deep space with different shielding by Monte Carlo simulations using the FLUKA code coupled to a mathematical model and a voxel phantom.

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(297) Furthermore, it is of interest to which extent the values of dose equivalent in these radiation fields depend on the different concepts of the quality factor. As function of Z of the primary incident particle with an energy distribution as given in the GCR field, Fig. 6.14 shows a comparison of body-averaged mean quality factors using the Q(L) relationship and the recent NASA approach to quality factors based on particle track structure concepts and using different quality factor values for solid cancers and leukaemia. The calculations are made using the HZETRN code for solar minimum in the orbit of the International Space Station (ISS). Different calculations were carried out applying either a thin (5 g/cm²) or a thick (20 g/cm²) aluminum shielding. Differences between the different approaches occur mainly for low- and high-Z particles.

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3776 Figure 6.14. Comparison of body-averaged quality factor values as a function of primary incident
 3777 particle of charge Z calculated by applying either the ICRP $Q(L)$ function or the NASA quality factors
 3778 for solid cancers or leukaemia for thin or thick aluminum shielding conditions. (A) 5 g/cm^2 Al
 3779 shielding; (B) 20 g/cm^2 Al shielding.

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3782 6.4 Assessment of doses in the body by measurements

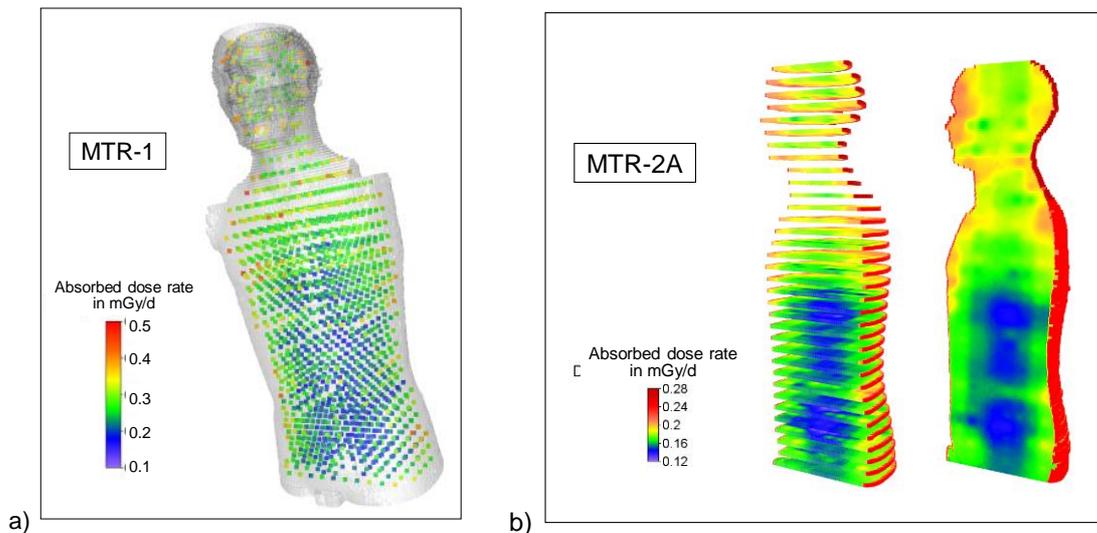
3783

3784 (298) Specific operational quantities for an assessment of dose in the body of astronauts in
 3785 space have never been defined. Some properties of the radiation field, however, may provide
 3786 some help in the determination of effective dose equivalents of astronauts. Firstly, there is a
 3787 major component of very high-energy particles with long ranges in tissue in the field which
 3788 strongly contribute to doses in the body. Secondly, the radiation incidence on the body may
 3789 be nearly isotropic considering the situation that the astronauts are usually moving around
 3790 and are not fixed in a special position for long times. Therefore, isotropic exposure of
 3791 astronauts has been assumed in calculations resulting in a relatively homogeneous dose
 3792 distribution within the body from GCR exposure (see Fig. 6.12 and 6.13). This, however, is
 3793 not the case for low-penetrating radiation which mainly contributes to doses of the skin and
 3794 other tissues near to the surface.

3795 (299) Any dose-measuring system, therefore, must have the ability to discriminate
 3796 between strongly- and low-penetrating radiations. In addition, information about the
 3797 distribution of absorbed dose in tissue in terms of lineal energy L , D_L , is necessary for the
 3798 assessment of equivalent dose in the body.

3799 (300) Measurement of dose distributions and organ doses in a human phantom in space
 3800 have also be performed by the MATROSHKA collaboration (Reitz and Berger, 2005; Reitz
 3801 et al., 2009) and exploited by members of the HAMLET collaboration (see www.fp7-hamlet.eu). A tissue-equivalent anthropomorphic phantom, called MATROSHKA, has been
 3802 equipped with hundreds of dosimeters of different types, mostly TLD and NTD - and
 3803 exposed in space during various ISS missions.
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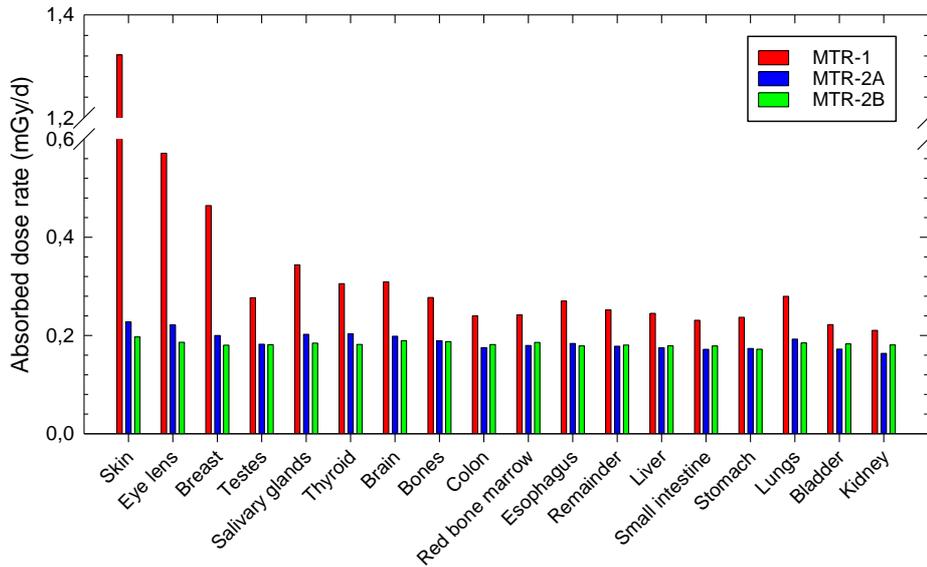
3805 (301) The MATROSHKA experiments provide detailed depth dose distributions in a
 3806 human phantom exposed outside the ISS (MTR-1 mission) and in two positions inside the
 3807 ISS at different shielding locations (MTR-2A, MTR-2B missions). Figure 6.15a shows the
 3808 measured dose rate distribution in the MATROSHKA phantom for the MTR-1 mission based
 3809 on TLD readings at 1598 locations inside the phantom. For measurements with the phantom
 3810 outside the ISS, the absorbed dose rates range from 0.1 mGy/d to 0.5 mGy/d, with the highest
 3811 dose at the phantom surface. For other missions inside the ISS due to the spacecraft shielding
 3812 the absorbed dose rates are mostly restricted to a range from 0.13 mGy/d to 0.23 mGy/d.
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 3814



3815 Fig. 15. (a) Measured absorbed dose rate distribution in the MATROSHKA phantom for the
 3816 MTR-1 mission outside of the ISS based on TLD readings (Reitz, 2012).
 3817 (b) Absorbed dose rate distribution in the MATROSHKA phantom for the MTR-A2 mission inside
 3818 the ISS obtained by interpolation of point doses over the whole phantom volume (Reitz, 2012).
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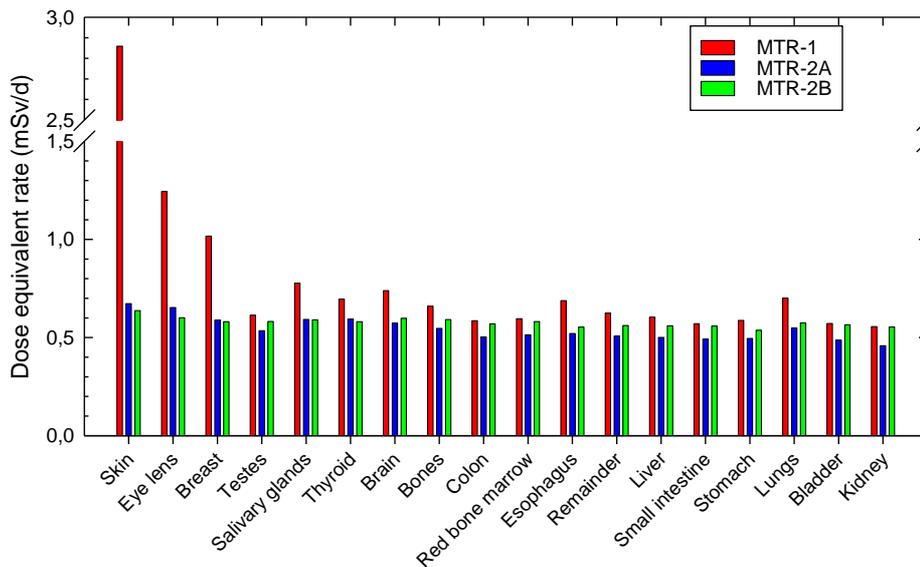
3821 (302) Computer tomography scans of the MATROSHKA phantom are taken in order to
 3822 build up a voxel model called NUNDO (Numerical Rando). A computer program allows
 3823 calculation of dose distributions by interpolation of point doses over the whole phantom
 3824 volume (see Fig. 6.15b). For total absorbed dose and dose equivalent rate distributions and
 3825 hence an assessment of mean absorbed dose rates and dose equivalent rates in organs and
 3826 tissues, data measured by TLD and PNTD have been combined. The dose equivalent rate was
 3827 calculated using quality factors as defined in ICRP Report 60 (ICRP, 1991).

3828 (303) The high dose gradient near to the skin for the MTR-1 missions is due to the high
 3829 contribution of electrons and protons at the South Atlantic Anomaly. Inside the ISS this
 3830 contribution is strongly reduced due to the shielding by the spacecraft (see Fig. 6.16 and
 3831 6.17). The dose rates in the deeper lying organs are nearly constant due to the high energies
 3832 and nearly isotropic fluence distribution of the GCR. For astronauts moving within the
 3833 spacecraft, this is even more the case.



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Fig. 6.16 Mean absorbed dose rates in organs and tissues of the MATROSHKA phantom during different space missions at the ISS determined from measurements using TLD and PNTD (Reitz, 2012).



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Fig. 6.17. Mean dose equivalent rates in organs and tissues of the MATROSHKA phantom during different space missions at the ISS determined from measurements using TLD and PNTD (Reitz, 2012).

(304) The measured organ absorbed dose data were compared with corresponding values obtained from dose conversion coefficients multiplied with cosmic-radiation fluxes in the spacecraft calculated by PHITS using a simplified geometry of the ISS (Sato et al 2011). The agreement was found quite satisfactory in spite of some discrepancies observed for some organs, taking account of the various approximations introduced, in primis the hypothesis of isotropic irradiation.

(305) Similar data have been calculated for comparison with organ absorbed doses measured by the MATROSHKA experiment outside the ISS. As shown in Table 6.3 an

3851 acceptable agreement has been achieved.

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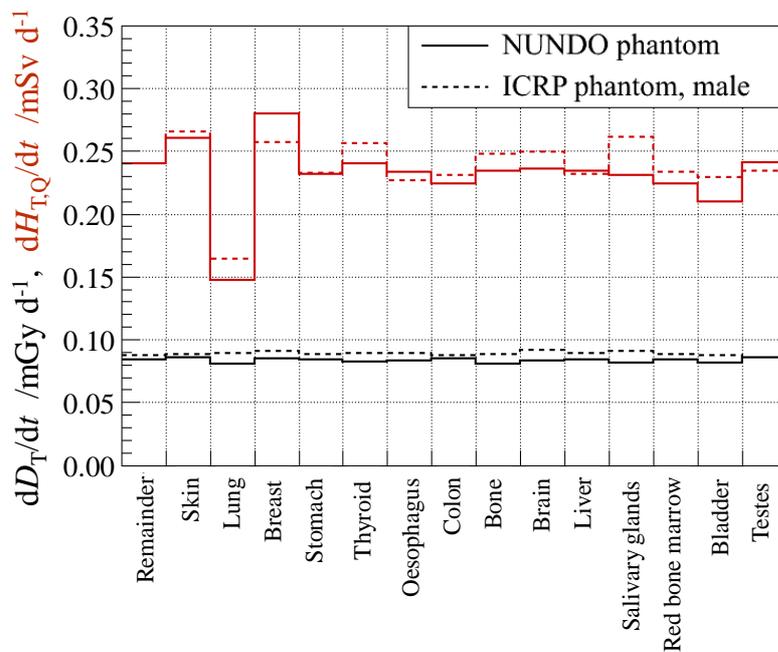
3853 Table 6.3. Organ absorbed dose rates measured by the MATROSHKA experiment outside ISS (Reitz
 3854 et al., 2009) in comparison with corresponding calculated dose rates obtained from PHITS
 3855 simulations (Sato et al., 2011).

Organ/Tissue	Measured absorbed dose rate mGy/d	Calculated absorbed dose rate mGy/d
Skin	0.944	1.814
Salivary glands	0.33	0.435
Breast	0.39	0.690
Lung	0.26	0.279
Oesophagus	0.24	0.250
Stomach	0.242	0.245

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3857 (306) The agreement is seen to be very good for the interior organs such as lung,
 3858 oesophagus and stomach. The calculation, however, substantially overestimates the measured
 3859 absorbed dose rates for the organs located near to the surface of the body, especially for the
 3860 skin. That is probably due to the effect of a high yield of low-energy trapped protons and
 3861 electrons encountered and also to some differences between phantoms and geometries of
 3862 their surrounding environment employed in the calculation and experiment.

3863 (307) The accuracy of heavy ion transport codes was discussed in Section 5.3.1. With
 3864 respect to the different anthropomorphic voxel models used the differences in the values of
 3865 absorbed dose and dose equivalent of single organs and tissues calculated using the NUNDO
 3866 model (MATROSHKA) or the ICRP reference voxel phantom are quite small as shown in
 3867 Fig. 6.18 (Matthiä, 2012).



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3869 Fig. 6.18. Mean absorbed dose rate and dose equivalent rate in various organs and tissues
 3870 calculated using the GEANT4 code and the NUNDO model and the ICRP reference phantom for
 3871 galactic cosmic radiation (Matthiä, 2012).

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3873 **6.5 Biodosimetric measurements**

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3875 (308) Biodosimetric measurements offer an alternative to the measurement of individual
3876 doses external to the body of an astronaut for the assessment of effective dose equivalent,
3877 because of the very complex radiation field in the spacecraft which in addition is varying in
3878 intensity and composition with time. Lymphocytes in the human body are circulating and
3879 hence provide a target which is well distributed over the human body. In addition, the RBE-
3880 LET dependence for total chromosomal exchanges during the first cell cycle is similar to the
3881 $Q(L)$ relationship defined by the Commission (see Fig. 3.4) when using the premature
3882 chromosome condensation method (Cucinotta et al., 2008).

3883 (309) The method has already been applied for estimating doses of astronauts in space
3884 shuttle flights (George et al., 2001), at the MIR station (Yang et al., 1997, Fedorenko et al.,
3885 2001) and at the ISS (Cucinotta et al., 2008).

3886 (310) In Table 6.4 doses obtained by physical and biological dosimetry for ISS missions
3887 are compared, where the comparison includes biomarker results based on an individual or
3888 population based calibration using gamma radiation (Cucinotta et al., 2008). Mission lengths
3889 of 4 to 7 months occur for the different results described. Biomarker results are given in
3890 terms of RBE-D (unit: mGy). This RBE-weighted dose value may be compared with values
3891 of dose equivalent to organs or tissues (here with the dose equivalent of the skin) and
3892 effective dose equivalent obtained from dosimeter readings and applying the $Q(L)$ relation as
3893 given by Publication 103 (ICRP, 2007). The overall agreement between the methods lends
3894 confidence that the complex environment in space has been adequately characterized.
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Table 6.4. Data of mission doses of ISS astronauts obtained by biological dosimetry and by measurements with individual dosimeters (Cucinotta, 2008). The data of skin dose equivalent and effective dose equivalent are obtained by calculations adjusted to the individual's dosimeter readings.

Astro- naut	RBE·D mGy		Individual dose- meter reading in mGy	Skin dose equivalent (calc.) mSv	Effective dose equivalent (calc.) mSv
	Individual based calibr.	Population based calibr.			
1	94 ±12	128 ±25	31	89.9	77.6
2	127 ±57	84 ±41	30	86.5	73.7
3	78 ±16	81 ±19	33	96.4	82.1
4	60 ±24	87 ±20	32	93.8	79.9
5	36 ±15	54 ±26	29	85.1	72.5
6	59 ±19	61 ±21	32	90.8	80.0
7	41 ±19	72 ±27	29	83.3	70.6
8	83 ±29	40 ±21	31	88.3	74.7
9	113 ±17	130 ±25	40	115	98.6
10	-	75 ±26	31	88.3	74.5
11	74 ±32	55 ±26	22	64.5	54.7
12	128 ±40	71 ±24	23	65.4	55.7
13	134 ±45	88 ±29	22	64.7	59.8
14	66 ±21	59 ±15	26	78.0	66.3
15	83 ±27	125 ±52	30	88.6	75.2
16	10 ±24	15 ±35	20	56.8	47.5
17	147 ±48	134 ±66	36	103.0	86.3
18	113 ±26	109 ±34	30	83.7	76.9
19	119 ±32	69 ±23	24	70.1	59.5
Mean*	85 ±38	81 ±32	28.9 ±4.9	83.8±14.1	71.9 ±12.0

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- Uncertainties listed are based on the data variation in the column and do not include measurement uncertainties.

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7. OPERATIONAL RADIATION PROTECTION IN SPACE

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(311) The guiding principles used for radiation protection on Earth including justification, dose and risk limitation and ALARA are essential also for radiation protection during space travel. Operational radiation protection for astronauts in space, however, differs significantly from those requirements for external radiation exposure on Earth (see for example EC, 2009). On Earth, doses are generally well below annual limits; if constraints are approached, better estimates of effective dose are made from the results of measurements of operational quantities. For astronauts, doses are mainly based on the environmental situation in space, being typically about 1 mSv per day. Radiation protection for missions includes a large range of different measures which should all have the aim of reducing the radiation exposure of astronauts to a level where the health risks are acceptable (NCRP, 1989). In contrast to many of the other flight risks, effects from radiation exposure can have long latency times, and cancer and other detriments may occur long after a space mission has ended. However, as noted by the NCRP (NCRP, 1989, 1997) and discussed by others (Schimmerling 2010; Cucinotta et al., 2011b), the acceptance of radiation risks in spaceflight should consider many factors both dependent and independent of other flight risks.

(312) The operational radiation protection for each mission is to assess the radiation exposure of astronauts in space. This is prospectively performed by calculating organ and tissue doses weighted for radiation quality and, if needed also effective dose equivalent considering tissue weighting, for comparison with mission dose or risk limits related to stochastic and tissue reactions. After the mission, all available data should be combined, including results from measurements of area and individual instruments, to assess the doses and the probability of a radiation induced event.

(313) There may be defined special dose levels or constraints for short term exposure, annual exposure, and for an astronaut's career. The dose levels should apply to the assessed total detriment, to the lens of the eye, and to the skin. The total detriment is related to a probability of cancer risk. The dose constraints for tissue reactions are to avoid the occurrence of impairment during or after a mission. The complex nature of the radiation field in space requires continuous analysis of the environment by calculation, area monitoring of the astronauts' environment, and, where possible, the analysis of the results of active personal dosimeters, in order to meet action levels and the dose limits.

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7.1 Pre-flight mission design

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(314) The flight mission design needs to be carried out with the aim of reducing radiation exposures in line with ALARA. The application of ALARA requires that the space radiation environment is well known – outside and inside a spacecraft. This requires knowledge of the external radiation environment with its various components (see Chapter 2), of the spacecraft/habitat construction, and of the results of transport calculations modelling the internal radiation environment.

(315) The design of the spacecraft/habitat requires the use of radiation transport codes to compute dose equivalents. As described in Chapter 5 the computer codes may be one- or three-dimensional, deterministic or based on Monte Carlo (MC) methods. The construction of a spacecraft/habitat should include areas where the dose rates are lower than elsewhere in the spacecraft. There should be area monitors with visual displays of dose rates.

(316) To reduce uncertainties (see 7.5) further improvements are needed in the models of

3953 the galactic cosmic radiation, the solar energetic particles, and the trapped radiation to allow
3954 the accurate forecasting of the fully integrated model of the radiation environment incident on
3955 the spacecraft/ habitat. Models have been developed for each of the radiation components.
3956 These models suffer several shortcomings: (i) the GCR models inadequately characterize the
3957 solar cycle dependency and the scaling with heliocentric distance; (ii) the SPE models have
3958 an incomplete understanding of the acceleration mechanism of the transport through the
3959 heliosphere and a lack of prediction capability; (iii) the radiation belt models no longer reflect
3960 the current state of the Earth's magnetosphere and lack the ability to properly describe the
3961 dynamic behaviour of the trapped particles.

3962 (317) The forecast models require an improved understanding of the physical processes on
3963 the Sun; the transport and acceleration of the solar wind through the heliosphere; the
3964 processes in the magnetosphere (wave-particle interactions, source and loss processes, and
3965 acceleration mechanisms). The space environment is highly variable on very different time
3966 scales as a result of the variability of the Sun.

3967 (318) In general all aspects of the space environment are affected, but SPEs and CMEs are
3968 the most dramatic radiation events and may constitute for several missions a serious hazard.
3969 All the radiation components (including GCR and trapped) are also modulated by SPEs
3970 (Forbush decreases in the GCR fluences, for example). An accurate prediction of SPEs and
3971 CMEs would allow for a more effective approach in the shielding strategy. Forecasting
3972 through real time observation and propagation modelling should be improved.

3973 (319) Astronauts are particularly vulnerable during EVAs, when they should be monitored
3974 with active dosimeters. Real-time space weather predictions and remote satellite and areas
3975 instrumentation will assist in EVA activity. The real-time measurements will provide
3976 guidance, and can suggest changes in mission scheduling to maintain the total risk below
3977 predefined limits.

3978 (320) The development of shielding requirements and strategies is important for the
3979 achievement of ALARA. The reduction in exposure can be made by reducing the exposure
3980 time or by passive shielding. Passive shielding may cause an increased risk by increasing the
3981 dose equivalent from any generated secondary particles, projectile and target fragments
3982 (including neutrons). For shielding effectiveness, the use of a shielding material with a low
3983 mean atomic mass is generally better.

3984 (321) Information about radiation transport codes (see Chapter 5) is important and the
3985 strengths and weaknesses of the codes should be investigated in detail via benchmarking
3986 procedures against experimental data, including data obtained with advanced
3987 anthropomorphic phantoms exposed at accelerators. The physics at the basis of the particle
3988 transport and cross sectional data tables must also be improved to further develop the codes.

3989

3990 **7.2 Area monitoring**

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3992 (322) Area monitors at well selected locations in the spacecraft can determine the
3993 environmental conditions and are appropriate for an immediate warning about changing
3994 exposure conditions.

3995 (323) Instruments are required to determine the radiation environment in terms of particle
3996 type, fluence rate, energy, and direction distributions and, in some instances, dose quantities.
3997 Dose quantities used to assess doses to astronauts and to monitor radiation at a number of
3998 locations should give values of the dose rate. These data can be used to implement ALARA.
3999 Area monitors at well selected locations in the spacecraft can be appropriate for immediate
4000 warning about changing exposure conditions. This can be of importance before or during

4001 SPEs, electron belt enhancements, and EVA. Real-time calibration of instruments should be
4002 explored.

4003 (324) If appropriately designed and accurately calibrated instruments are used, it may be
4004 that a quantity measured in fixed position in a spacecraft can, along with appropriate
4005 occupancy data, provide the basis for an adequate assessment of doses to an astronaut or of
4006 doses to the local skin or the extremities. While in principle this procedure may be applicable
4007 to astronauts in space, the large variation of the radiation field in intensity and composition of
4008 radiation types inside a spacecraft, and its variation with time together with the flexibility of
4009 the astronaut's position, has the consequence that area monitoring is not sufficient to
4010 completely substitute individual monitoring, especially considering the high individual doses
4011 to astronauts and the interest in providing a basis for individual risk estimates.

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4013 **7.3 Individual monitoring**

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4015 (325) The assessment of organ and tissue absorbed doses, together with radiation quality
4016 factors, of individual astronauts can be accomplished by calculations using anthropomorphic
4017 phantoms or by measurements using personal dosimeters (see Chapter 6).

4018 (326) One method of calculation of organ and tissue absorbed doses and radiation quality
4019 factors does so directly for a standard male or female phantom for various locations in a
4020 spacecraft with appropriate shielding. The phantoms can be adjusted to approximate a
4021 particular astronaut. The results are normalized using readings of area monitors and personal
4022 dosimeters. Another method requires knowledge of particle fluence and applies conversion
4023 coefficients from particle type, energy and direction distribution of fluence to organ and
4024 tissue absorbed doses and corresponding radiation quality factors for uniform irradiation of
4025 an astronaut.

4026 (327) Individual monitoring is mostly performed using personal dosimeters worn at the
4027 surface of the body. The personal dosimeter serves as the dosimeter of record. A single
4028 dosimeter system is, however, not sufficient to provide an assessment of the absorbed dose at
4029 the surface of the body weighted by radiation quality. The broad range of different types of
4030 particle requires at minimum two detectors, one sensitive to low-LET radiation and the other
4031 to high-LET radiation. Because of a possible anisotropy of the exposure in the spacecraft due
4032 to variations of shielding properties, it may be useful to wear more than one dosimeter. Also
4033 care needs to be taken regarding low-energy electrons and particles which are stopped in the
4034 skin and, therefore, contribute only marginally to organ doses other than the skin dose, but
4035 may induce a large signal in an external dosimeter.

4036 (328) The use of adequate active personal radiation detectors would enable improved
4037 characterization (input energy, nuclear abundance, fluence rate, direction) of the radiation
4038 field both on the body of the astronaut as well as in the environment. The measurement of
4039 dose-rate can contribute directly to ALARA.

4040 (329) The results of bio-marker measurements can be additionally used to estimate
4041 individual radiation exposure. The determinations can be collaborative and provide all the
4042 experimental radiation information and relative codes needed to achieve an efficient risk
4043 assessment, minimizing the uncertainties in the final risk estimates.

4044

4045 **7.4 Dose recording**

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4047 (330) Astronauts in space are exceptionally exposed and the assessment of their individual
4048 doses should be part of the radiation protection programme for space flights. Astronauts

4049 should be informed of their doses and risk assessments as soon as possible. Their doses
4050 should be regularly registered and a long term registry for all missions should be maintained.

4051 (331) The dose record is the formal statement of the crew member's exposure and should
4052 be kept as a confidential medical record. The record should contain the history of the
4053 exposure and all the calculation and experimental results, including all information on the
4054 particle type energy and direction distributions of fluence; computer codes; conversion
4055 coefficients and weighting factors; area monitor, personal dosimeter, and biomarker results

4056

4057 **7.5 Consideration of uncertainties**

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4059 (332) There are large uncertainties in projecting cancer risks and the risks of other late
4060 effects from ionising radiation on Earth. Space radiation carries additional considerations,
4061 which further increase uncertainties. As radiation workers approach a significant fraction of
4062 exposure limits, the calculations of uncertainty bounds is needed because exposures leading
4063 to acceptable levels of risks may no longer be confidently avoided when the uncertainties are
4064 considered.

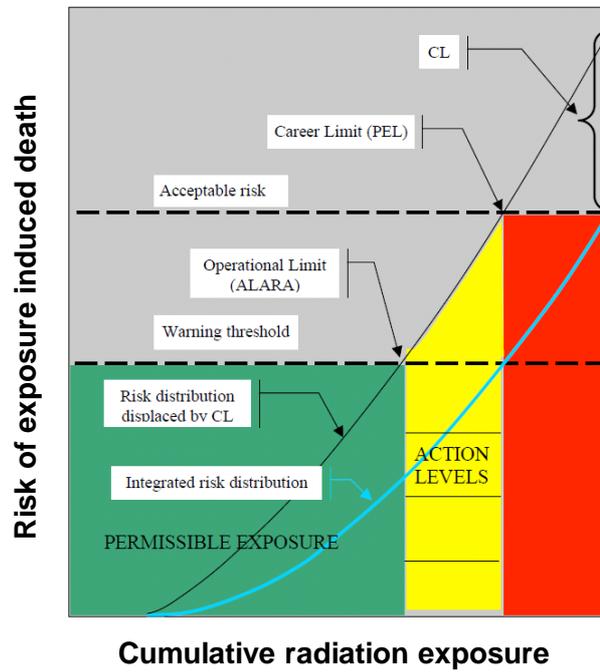
4065 (333) It is important to provide all the information needed to establish the uncertainties of
4066 the organ risk estimates. The overall uncertainty will be reduced by improvements in
4067 modelling of radiation sources and the transport of the field through the spacecraft or habitat;
4068 improved accuracy of radiation transport codes; improved accuracy of radiation monitoring;
4069 better estimation of dose and dose-rate effectiveness factor; better determination of radiation
4070 quality factors; better understanding of the statistics and dosimetry of epidemiological data.

4071 (334) A full risk model may need to be developed using real time radiation readings, space
4072 weather forecasts, and risk assessments. This can allow changes to the mission, with an
4073 assessment of uncertainties, whilst maintaining the total risk below predefined limits.

4074 (335) Uncertainties in estimates of exposures and the relationship between exposure and
4075 risk are a major concern for operational radiation protection in space due to the types of
4076 radiation, which includes heavy ions and neutrons, and the higher exposure levels, which
4077 may approach exposure and risk limits. Figure 7.1 illustrates schematically an operational
4078 approach where an evaluation of uncertainties is included in the radiation protection approach
4079 (Schimmerling, 2010). Obviously, "acceptable" levels of cumulative exposure depend on the
4080 uncertainty and hence reduction of uncertainties is seen to be an important task for mission
4081 planning. Methods to estimate uncertainties in exposure and risks are described elsewhere
4082 (Cucinotta et al., 2011; NCRP 2006)

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Fig. 7.1. Schematic view of risk management with ALARA and large uncertainties. The risk distribution function versus cumulative radiation exposure with and without considering uncertainties (based on 95% confidence limit (CL)) is displaced. ALARA practices and action levels for an “acceptable” level of risk and permissible exposure limit (PEL) are also shown (Schimmerling, 2010).

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4092 **8. CONCLUSIONS**

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4094 (336) Astronauts in space are living under extraordinarily different environmental
4095 conditions than on Earth. The radiation environment in space needs specific attention and
4096 considerations regarding the exposure of astronauts and the limitation of such exposures to a
4097 level where the health risks are comparable to other risks during space missions.

4098 (337) The number of astronauts undergoing missions in space is very small compared to
4099 the number of occupationally exposed persons on Earth. In long-term missions, however,
4100 doses to the astronauts are generally higher than those at other working places on Earth.
4101 Hence a more individually based dose and risk assessment should be performed for
4102 astronauts in space compared to persons on Earth, where these exposures are usually much
4103 less than the limits defined for occupationally exposed persons.

4104 (338) The following points are specifically mentioned:

4105 • The primary radiation field in space is complex and includes electrons, protons,
4106 α particles and heavy ions up to very high energies. Additional secondary radiation
4107 (e.g. γ -radiation, muons, neutrons and pions) is produced by interactions in the
4108 materials of a spacecraft, its equipment, and in the astronauts.

4109 • The physics at the basis of the particle transport and cross sectional data tables must
4110 be improved to further develop the computational methods. There is a lack of
4111 experimental cross-section data for light fragments and neutrons. Codes need to be
4112 improved to treat all primary and secondary cascades including photons, protons, light
4113 ions, heavy ions, mesons and electromagnetic cascades. The nuclear interaction
4114 database needs to be updated, especially for neutrons and light ions.

4115 • The simple concept of considering the differences in radiobiological effectiveness by
4116 radiation weighting factors, w_R , e.g. a constant radiation weighting factor of 20 for all
4117 heavy ions of all energies, is not appropriate for dosimetry in space and the quality
4118 factor (Q) is applied for the definition of the quantity dose equivalent in an organ or
4119 tissue of the human body.

4120 • The basis for risk assessments for the astronauts are the dose equivalents in organs
4121 and tissues of adult males and females, $H_{T,Q}^M$ and $H_{T,Q}^F$, which are based on mean
4122 absorbed doses, D_T , and mean quality factors in the corresponding organs or tissues,
4123 Q_T .

4124 • Conversion coefficients which relate particle fluence to mean absorbed doses in
4125 organs and tissues of the human body and corresponding mean quality factors for all
4126 types of radiation present in space, are an important data base for the assessment of
4127 the exposure of astronauts. For the estimation of radiation risks of astronauts based on
4128 mean absorbed doses in the body, an assessment of the uncertainty of D_T and Q_T
4129 would be very useful.

4130 • The concept of operational dose quantities for area monitoring of external exposure
4131 and an assessment of effective dose is not applicable because many different types of
4132 particles are involved with very high energies. Instead the measurement and
4133 determination of particle fluence and its distribution in energy and direction is more
4134 important and provides a basis for an assessment of doses.

4135 • A broad range of instrumentation has been specifically designed for fluence and dose
4136 measurements in space. Obviously a single instrument is not sufficient for a
4137 determination of all particle fluences and their energy distributions and for an

- 4138 assessment of organ doses in the human body. Particle spectrometers, individual
4139 dosimeters and specific instruments measuring the low-penetrating radiation on the
4140 body of an astronaut are needed.
- 4141 • While passive individual dosimeters are generally appropriate to measure mission
4142 doses integrated over the flight time, active detector systems allow the measurement
4143 of fluctuations in the exposure and also the inclusion of warning capabilities, e.g. in
4144 cases of a large SPE.
 - 4145 • The use of biomarkers of health effects is an attractive supplement to physical
4146 dosimetry. Biomarkers from blood samples from astronauts can be used to assess
4147 individual mission doses. In addition, biomarkers may serve for an individual risk
4148 assessment or an indication for the presence of a disease. Biodosimetry is routinely
4149 performed on the small population of astronauts involved in International Space
4150 Station (ISS) missions, and would likely be used in future space missions. To be
4151 useful as a biodosimetry method, the radiation quality dependence of the response of
4152 the specific assay need to be known.
 - 4153 • The exposure of astronauts in space is a special case of environmental exposure quite
4154 different from that on Earth. In space missions, especially in long-term interplanetary
4155 missions, their exposure will be higher than the annual limits recommended for
4156 exposure of workers on Earth. Although astronauts are exposed to ionizing radiation
4157 during their occupational activities they are usually not classified as being
4158 occupationally exposed in the sense of the ICRP system for radiation protection. Thus,
4159 for a specific mission planned, reference levels for risks or doses may be selected at
4160 appropriate levels and no dose limits may be applied for a given mission.
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5021 ANNEX

5022

5023 **Conversion coefficients and mean quality factors for protons, neutrons, alpha particles**
5024 **and heavy ions ($2 < Z \leq 28$).**

5025

5026 (1) Conversion coefficients for mean absorbed doses in organs and tissues for males and
5027 females based on the reference voxel phantoms (ICRP, 2009) are given for heavy ions
5028 ($2 < Z \leq 28$) with energies from 10 MeV/u to 100 GeV/u. For the skin data are given for the
5029 energy range from 1 MeV/u to 100 GeV/u. The exposure geometry is always isotropic (ISO)
5030 radiation incidence. The data were evaluated based on their calculated values using the
5031 PHITS code (Sato et al., 2010).

5032 (2) Based on the $Q(L)$ function (ICRP, 1991), also mean quality factors for organs and
5033 tissues of the adult male and female reference phantom are given for the heavy ions and
5034 energies mentioned above (Sato et al., 2010).

5035 (3) In addition to the mean quality factors based on $Q(L)$, data are presented for mean
5036 quality factors for organs and tissues of the adult male and female reference phantom based
5037 on the Q -concept developed by Cucinotta et al. (Cucinotta et al., 2011a) (see 3.2.3). The data
5038 were also evaluated based on their calculated values using the PHITS code (Sato et al.,
5039 2012). The quality factors are related to RBE for solid cancer, and those for leukaemia are
5040 one-fourth of the corresponding data for solid cancer.

5041 (4) Furthermore, conversion coefficients from ICRP Publication 116 (ICRP, 2012) for
5042 isotropic (ISO) exposure and mean quality factors are given for protons, neutrons, and alpha
5043 particles. The mean quality factors were evaluated based on their calculated values using the
5044 PHITS code (Sato et al. 2009, 2010 and 2012). The energy range considered for these
5045 particles are the same as in ICRP Publication 116.